Maine Yankee
RELIABLE ELECTRICITY FOR MAINE SINCE 1972

EDISON DRIVE • AUGUSTA, MAINE 04330 • (207) 622-4868

June 6, 1991
MN-91-87 SEN-91-161

Region I
UNITED STATES NUCLEAR REGULATORY COMMISSION
475 Allendale Road
King of Prussia, PA 19406

Attention: Mr. Thomas T. Martin, Regional Administrator

References:
(a) License No. DPR-36 (Docket No. 50-309)
(b) USNRC Letter to Maine Yankee dated March 27, 1989 - NRC Safety System Functional Inspection Team Report No. 50-309/89-80
(c) Maine Yankee Letter to USNRC dated June 13, 1989 (MN-89-80) - Response to NRC Safety System Functional Inspection Team Report
(d) Maine Yankee Letter to USNRC dated August 30, 1989 (MN-89-112) - Update
(e) Maine Yankee Letter to USNRC Region I dated January 15, 1990 - Status Update
(f) USNRC Letter to Maine Yankee dated March 15, 1990 - NRC Safety System Functional Inspection Follow-Up Team Report No. 50-309/90-80
(g) Maine Yankee Letter to USNRC dated April 20, 1990 (MN-90-43)- Response to SSFI Follow-Up Team Inspection Report 90-80
(h) Maine Yankee Letter to USNRC dated February 6, 1991, (MN-91-28) - Follow-Up Status to SSFI Inspection Report 50-309/90-80

Subject: Status Update - NRC Safety System Functional Inspection Team Report No. 50-309/89-80 and Follow-Up Team Inspection Report 90-80

Gentlemen:

In early 1989, an NRC Team conducted a Safety System Functional Inspection (SSFI) at Maine Yankee. Reference (b) reported the results of the inspection. Maine Yankee responded to the Inspection Report with Reference (c) and updated two items with Reference (d).

In early 1990, an NRC Team conducted a SSFI Follow-Up Inspection at Maine Yankee. Reference (e) updated Attachment 2 of Reference (c) to reflect the status of issues as of January 5, 1990. Reference (f) reported the results of the inspection. Maine Yankee responded to this Inspection Report with Reference (g) and updated two items with Reference (h).

IE01
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In anticipation of your upcoming Electrical Distribution System Functional Inspection, we have updated the Attachment to Reference (e) to reflect the status of issues as of about 06/01/91. This updated issues summary is attached.

Should you have any questions on the attached information, please contact us.

Very truly yours,

S. E. Nichols, Manager
Nuclear Engineering and Licensing

WBD/sjj

Attachment

c: Document Control Desk
Mr. E. H. Trottier
Mr. Charles S. Marschall
Mr. A. L. Della Greca - NRC Region I

ATTACHMENT

Priority and Schedule for Commitments
[Made during SSFI/Follow-up SSFI
(As response to specific issues in the Inspection Report)

The matrix which follows documents Maine Yankee's response to the specific [issues raised in the NRC SSFI/Follow-up Team Reports. The issues are listed (as Description) in roughly the order as presented in the reports.

Item Numbers were assigned for ease of cross-reference and tracking purposes; these numbers should not be confused with Action Item Numbers assigned and tracked during the course of the actual inspection.

Description is a restatement of the issue documented in the report. Reference to the report paragraph(s) which discuss(es) the issue is included.

The Plans column provides Maine Yankee's current plans for response to the issue.

The Status/Comments column includes indication of high priority items, some status information, and/or actions taken.

During the course of the inspections, Maine Yankee actively maintained an issues tracking document, "SSFI Audit Action Item Status Summary." The document was used to assure that each issue perceived by Maine Yankee was documented and responded to. Where appropriate, Maine Yankee committed to appropriate follow-up items. Although the attached matrix differs in format, sequence, etc., from that developed during the inspection, we have attempted to reflect all of our prior commitments in the attached.

An asterisk (*) in the Scheduled Completion column indicates that the item has been rescheduled to a date different from that provided in the original response.

[A "[" in the left margin is indicative of a change since the previous response - in the Scheduled Completion column or the Status/Comments column.

NRC SSFI AUDIT FINDINGS SUMMARY

PCC/SCC SYSTEMS

NRC REPORT NOS. 50-309/89-80 & 90-80

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
1	Design deficiencies associated with the reach rods for valves SCC-M-165 and PCC-M-43 have resulted in recurring failures to open. NRC Report Para. 3.1.1 5.4.3 3.3.6 3.1.4 (Refer also to Item No. 5)	Modify closing circuits to close on limit; install midspan bearing guides.	Complete	Upgrades completed.
2	A documented, up-to-date system heat balance is not available. NRC Report Para. 3.1.2	Contract S&W to perform and document a design basis heat load study.	Complete	Stone and Webster Engineering Corp. performed. Considering increasing flow to Diesel Generators.
3	RHR heat exchanger thermal performance has not been verified by test. NRC Report Para. 3.1.2	Investigate means of verifying the adequacy of the RHR and CCW heat exchanger performance levels to meet design basis requirements.	Complete*	Per generic letter 89-13: 1) RHR performance data taken during 1990 refueling. Additional calculations required to evaluate data. 2) Inspections and/or test data collection performed on other safety related heat exchangers during 1990 refueling (report not yet issued). 3) Service water heat exchanger test program being implemented per G.L. 89-13 response.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
4	SCC trip valves accumulator tank design and capacity may not be sufficient to hold the trip valves closed for the required duration during accident conditions if the instrument air system has failed. NRC Report Para. 3.1.3 (Refer also to Item No. 29)		Complete	A temporary air source was installed during the audit. The calculation was revised and operational changes were recommended. See item nos. 29, 51A & 51B to track recommended operational changes.
5	The reach rod assemblies for PCC-M-43 and SCC-M-165 are not shown on controlled drawings; therefore, a change to roll pin material had not been well documented for future reference. NRC Report Para. 3.1.4 (Refer also to Item No. 1	As part of the modifications described in item #1, a controlled drawing for these reach rod assemblies will be referenced in the vendor drawing file.	Complete	Controlled drawing developed.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
6	The following check valves that perform safety functions are not included in test or inspection programs: PCC-508, 509 PCC-445, 446 NRC Report Para. 3.1.5	Include these valves in the IST Program. Also include PCC-72 and SCC-4, from Item 43.	Complete	NRC Inspection Report stated that test procedures will be included in the program for critical check valves. M.Y. committed to include these valves only in the IST Program. [Valves in IST Program submitted in response to NRC Generic Letter 89-04]
7	Calculation EDCR 80-45-CAL-1 for SCC isolation valves closure time did not consider thermal shrink/swell of the water inventory. Simultaneous surge tank vent valve closure should also be considered. NRC Report Para. 3.1.6	Contract S&W to perform the required calc. after the CCW heat balance is complete and system temperatures are defined.	Complete	Preliminary review completed during the inspection indicated that system operability will not be adversely affected. Confirmed by recalculation.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
8	EOPs do not provide adequate guidance concerning service water cross-tie lineup. NRC Report Para. 3.1.7 (Refer also to Item No. 35)		Complete	Step 16 of EOP E-0 was revised to require isolation of both service water heat exchangers in the inoperable CCW system if only one service water pump is available.
9	Sufficient motor data not available to verify if AC overcurrent protection is conservative. NRC Report Para. 3.2.1.1	Establish a schedule to draft a motor overload protection setpoint manual, which will provide the setpoints and setpoint bases as a reference manual.	03-31-92*	Discussed the basis for overcurrent protection setpoints with S&W and we both believe that the technical basis is valid. Refer to response to Executive Summary Issue (3) of reference (c) for further discussion. Development of the manual is scheduled. S&W to provide thermal damage curves by 12/31/91. Coordination curves to be generated and Relay Data Base to be completed by 03/31/92.
10	MOVs may have oversized thermal heaters. NRC Report Para. 3.2.1.3	No new committed actions	Complete*	Recalculating overload heater sizes was already being addressed by Maine Yankee as part of our ongoing MOV project.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
11	DC short circuit calculation, E-1, has not been updated to reflect the new battery. Miscellaneous errors in calc. MYC-104. NRC Report Para. 3.2.2.1, 3.2.3.2, 3.2.4.1(a)	Revise DC short circuit calculation E-1 and battery sizing calc. MYC-104 to reflect the new batteries and correct errors.	Complete	Calculations revised to reflect new batteries.
12	The existing DC short circuit calculations show a fault current of 13,000 amps but the DC breakers are only rated for 10,000 amps. NRC Report Para. 3.2.2.1	Modify electric cable run from batteries 1&3 to busses 1&3	End of 1992 Refueling Outage* (begins Feb. 1992)	Modify electric cable run from batteries 1 and 3 to busses 1 and 3.
13	The AC short circuit calc. did not consider variations from nominal 345 KV. NRC Report Para. 3.2.2.2	Include the effect of voltage variations in the AC short circuit study.	Complete*	Response to NRC concern provided by MN-90-43.

ITEM NUMBER	DESCRIPTION	PLAN	SCHEDULED COMPLETION	STATUS/COMMENTS
14	The D-G voltage recovery curve came from the D-G proposal and was not verified after the equipment was purchased. NRC Report Para. 3.2.3.1	Contact GM and evaluate the adequacy of the D-G voltage recovery curve included in Calc. MY-107.	Complete*	MK letter of December 18, 1989 confirms adequacy.
15	The D-G loading calc. failed to evaluate the effect of multiple starts on individual motor control center loads. Thermal overload protection could also be affected. NRC Report Para. 3.2.3.1	Calc. MY-107 will be revised for this concern during the next scheduled review of this calc.	Complete*	Response to NRC concern provided by MN-90-43.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
16	Batteries 2 & 4 were replaced as a component substitution instead of as a design change NRC Report Para. 3.2.4.1(b) 7.1 7.2 7.5 5.1(b) (Refer also to Item Nos. 31, 48 and 50)		Complete	Developed a new procedure to improve component substitution evaluations using EPRI guidance. See #48 for additional actions.
17	Batteries 2 & 4 were tested at the wrong discharge rate both at the manufacturer's plant and at Maine Yankee. NRC Report Para. 3.2.4.1(c) (Refer also to Item No. 53)	Re-evaluate MY's and manufacturer's test data for batteries 2&4 using the correct test curves. Revise the test procedure prior to the next scheduled test.	Complete*	Satisfactorily reevaluated. Procedure revision complete.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
18	The component numbering system for the electrical schematics is different (S&W nos.) then the plant equipment numbering system (high no. FMs). NRC Report Para. 3.2.4.2		Complete	We have considered this and have concluded both systems are beneficial to Maine Yankee and will continue to be used.
19	Maine Yankee Document Control does not maintain either a copy of YAEC calculations or a list of calculations. The YAEC list of calculations does not indicate the latest revision of each calc. NRC Report Para. 3.2.4.3		Complete	A copy of the YAEC list of calculations has been distributed to the Maine Yankee site. This list shows current revisions of calculations. List is updated and reissued monthly.
20	A formal method of reviewing electrical load changes does not exist for changes other than those controlled by a safety-related EDCR. NRC Report Para. 3.2.4.4.		Complete	A formal method is (was) in existence which accomplishes review of EDCRs (safety-related and NNS) for electrical load changes; this method was apparently not explained well enough during the inspection. Maintenance (i.e., component substitution) should not affect electrical loading, however, we have revised our component substitution process to ensure component substitutions are reviewed for load change effects.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
21	AOP 2-36 states that DC low voltage alarm is 120 volts while the electrical training document PGS-18, Chapter 33, states on pages 116 & 129 that the low voltage alarm is 115 volts. NRC Report Para. 3.2.4.5(a)		Complete	The AOPs govern plant operation. Training manuals are updated periodically but disclaimer in the manual states that the manuals are not to be used to operate the plant.
22	Alarm response procedures do not include specific actions in response to 4000 volt motor overloads (CCW pump motors). Stone & Webster guidance states that motors should be shutdown manually as soon as possible. NRC Report Para. 3.2.4.5(b)		Complete	We reviewed AOP 2-37, Panalarm Response, and we find that while there is not a specific response unique to each of the 23 4160 V motors, there is a specific response to a motor trip, over-current, or thermal overload condition for any 4160 V motor. We believe this is appropriate and we do not plan any further action on this issue.
23	Technical Specifications do not require either battery to be in service below 210°F/ 400 psig. NRC Report Para. 3.2.5.1	Procedures will be revised as indicated in response to Executive Summary Issue (2).	Complete	This item is specifically addressed in response to Executive Summary Issue (2). Procedures revised.

PCC/SCC SYSTEMS

NRC REPORT NOS. 50-309/89-80 & 90-80

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
24	Technical Specifications do not include restrictions (LCO's) for operating with DC busses crosstied. NRC Report Para. 3.2.5.2	Procedures will be revised as indicated in response to Executive Summary Issue (2).	Complete End of 1992 Refueling Outage* (to begin Feb. 1992)	1. Operations procedures changes per MN-90-43. 2. Install a spare battery charger and inverter.
25	No formal documentation of set point basis. NRC Report Para. 3.3.1	Develop a set point basis document.	Complete	Setpoint basis document issued for Technical Specification Tables 4.1-1, 4.1-2, FSAR App. D, Table 5.10 and Class 1E circuits which have an automatic safeguards function.
		Implement a setpoint control program for the basis document.	07/01/91*	1. Revise plant procedures to provide control for changes affecting setpoints.
			07/01/91*	2. Develop an Instrument Setpoint Document Control Manual.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
25 (Cont'd)		Implement appropriate procedural controls for change of setpoints.	Complete	I&C procedure requires Engineering review of setpoint changes. Further controls to be developed under preceding item.
		Implement a setpoint control program for technical validation of setpoint changes.	Complete	New Engineering technical evaluating procedure addresses.
		Revise calibration procedures to agree with loop accuracy calculations.	12-31-91	To be revised during the procedure review cycle subsequent to December 31, 1989. Includes all procedures listed in the setpoint basis document.
26	The instrument calibration method is by calibrating components serially instead of testing the complete instrument loop. NRC Report Para. 3.3.2	Modify calibration procedures, for Technical Specification and non-Technical Specification Class 1E instruments as identified by Engineering, to incorporate a two-step loop calibration technique.	End of 1992 Refueling Outage*	List of instruments and additional information needed for procedure upgrade has been provided by Engineering. To be completed in conjunction with the procedure revisions discussed in Item 25.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
27	Maine Yankee does not have a formal program to review as found conditions during instrument PM activities. NRC Report Para. 3.3.3	Revise the PM Program to provide a formal review of as found conditions when performing calibrations.	Complete*	Procedure No. 6-03-4.2, Instrumentation & Controls Preventive Maintenance Activity developed. This procedure requires the I&C supervisor to review the as-found data in accordance with Instrumentation & Controls Department Performance Assessment Program, PAM-8. Any as-found condition outside the acceptance criteria is discussed with the shift supervisor for operability determinations.
28	Containment pressure transmitter PT 212 has a possible error greater than its significant reading in parts of its range. NRC Report Para. 3.3.4		Complete	The range was acceptable to NRC; refer to Section 3.3.5 of NRC letter to Maine Yankee dated 11-15-88 - Conformance to R.G. 1.97 accuracy. We agree with this conclusion.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
29	The low pressure alarm of 80 psig for TK-110 coincides with the minimum allowable pressure required by design basis calcs. NRC Report Para. 3.3.5 (Refer also to Item No. 4)	Raise the alarm set point to provide margin per memo MYP 89-305.	Complete	Setpoint changed during audit to provide margin. Final setting based on new calculation per MYP-89-305.
30	Valve PCC-M-219 was purchased in 1982 with a four rotor assembly but was wired as a two rotor assembly. NRC Report Para. 3.3.6	Evaluate the setup of PCC/SCC MOV's to determine if backfit is required (switch settings consistent with program developed for IEB-85-03) and backfit as necessary.	End of 1992 Refueling Outage*	PCC-M-219 will be rewired for four rotor operation during next outage.
31	Extensive modifications were made to the plant radiation monitoring system using the component substitution process instead of the design change procedures. NRC Report Para. 3.3.7, 7.2 (Refer also to Item Nos. 16, 48 and 50)		Complete	Developed a new procedure to improve component substitution evaluations using EPRI guidance. See #48 for additional actions.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
32	The informal DR log kept in the control room was not entirely up to date and correct. NRC Report Para. 4.1		Complete	NRC report stated that Maine Yankee agreed to implement procedures to maintain the log current. This list is a sort of the Plant's master DR listing, for items in the Control Room. The list is marked up by hand, shift-to-shift, to keep the list as current as possible between formal revisions, which are provided weekly. We have reviewed this and believe this practice should continue. (This item was not included in the SSFI Audit Action Item Summary which Maine Yankee maintained during the inspection to reflect commitments.)
33	Many valve tags were difficult to read, identifying as an "issue warranting further management attention" NRC Report Para. 4.1, 3.1.7, 4.5	Improve tags on the service water valves identified by the NRC and check for and correct other similar conditions.	Complete	A total of 153 valves are directed to be operated by the EOPs. Special labels installed.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
34	Operating procedures (AOP or Response Book) do not provide adequate operator guidance for response to surge tank high level (in-leakage). NRC Report Para. 4.2.2		Complete	Revised Alarm Response 2-36, AOP 2-25, and AOP 2-10 to provide more specific guidance for radiation monitor response and RCS in leakage.
35	Need documented evaluation to determine the impact on design basis analysis of operation with service water trains crosstied and service water flow through all CCW heat exchangers during a DBA. NRC Report Para. 4.2.3 (Refer also to Item No. 8)	Assemble the necessary documentation during effort to provide a Design Basis Summary Document.	Complete*	Emergency procedures were changed to isolate the idle train's CCW heat exchangers prior to a recirculation actuation signal (RAS). An analysis of operating the CCW system with one service water train supplying flow to all four CCW heat exchangers was performed.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
36	AOP 2-32 and AOP 2-33 did not establish the alternate fire main flow path to the affected D-G upon loss of CCW. NRC Report Para. 4.3.1		Complete	Revised AOP 2-32 and AOP 2-33 to provide all valve positioning required to align diesel cooling and to reflect the modification that switched cooling water supplies to AC-1A, 1B.
37	PM program for batteries 2 and 4 does not include a quarterly torque check of the terminal connections. NRC Report Para. 5.1.(b) and 5.1.(c).		Complete	PM E-5-Q-D does provide instructions for tightening to 125 in-lbs for batteries 1, 2, 3, & 4.
38	PM Program listed the equalizing charge time as 36 hours for batteries 1 & 3 and 24 hours for batteries 2 & 4 instead of the procedurally required 74 hours. NRC Report Para. 5.1(c)		Complete	PCR #88-256 dated 12-16-88 was previously issued to revise Surveillance Procedure 3.5.14 due to the battery replacement. Battery PM E-5-A-A has been revised for the correct equalizing charge for batteries 2 & 4.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
39	Maintenance issued a PCR to the P-10B maintenance procedure that eliminated a technical requirement without PED review. NRC Report Para. 5.2.1		Complete	The technical change was appropriate. Provided training to Maintenance Department supervisors concerning changes to procedures. PED now represented on Plant Operations Review Committee Procedure Review Subcommittee.
40	Control room panels contain a layer of dust. NRC Report Para. 5.3	Develop a PM activity to clean the inside of the main control board during refueling.	Complete*	NRC noted that the "Low Voltage Electrical Distribution Closeout Plan" action items to develop a control panel cleaning program has not been done. Panel cleaning completed. PM program updated per C&L requirements.
41	Control room panels contain a layer of dust. NRC Report Para 5.3	Evaluate the need for inlet filters on the MCB.	Complete	Evaluation complete. Inlet filters not necessary due to other changes.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
42	Improperly terminated and unidentified leads in the back of control room panels. NRC Report Para. 5.3		Complete	The specific discrepancies have been corrected. Other similar deficiencies will be detected and corrected during walkdown of control room electrical circuits previously planned and authorized.
43	PM program is different for valves PCC-72 and SCC-4 although the valves have identical functions. NRC Report Para. 5.4.1		Complete	Evaluated PM Program requirements for both check valves PCC-72 and SCC-4. Deleted PM since no reason for it was found. PM Program has been revised to provide better tracking of additions and deletions to the program. Valves to be included in IST Program; see Item #6.
44	Record keeping for accumulator check valve testing and DG testing was disorderly. SERP should be revitalized. NRC Report Para. 6.5.2		Complete	Added to program in 1987; two sets of data collected and included in SERP manual; records have been reorganized and will be maintained in an orderly fashion.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
45	PED procedure for preparing safety evaluations does not establish a standard content and format. NRC Report Para. 7.1 7.4		Complete	Procedure 17-21-7 provides a content checklist for safety analyses and leaves the format to the discretion of the writer. Past experience indicates that the safety analysis procedure is adequate.
46	No procedure for support groups to provide issues to the NSE group for evaluation of reportability, root cause analysis, and Human Performance Evaluation System study. NRC Report Para. 7.1, 7.4	Evaluate mechanisms for plant support groups to notify the NSE group when potentially reportable issues are identified.	Complete*	Procedure No. 20-3-1, Commitment Tracking System, was revised to establish a formal program to track all commitments made.
47	FSAR was not updated to reflect the new PCC/SCC heat exchangers. NRC Report Para. 7.1	Update the FSAR to reflect the modification of the two heat exchangers.	Complete	FSAR updated.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
48	Procedures for component substitution do not adequately prevent unauthorized design changes. M.Y. has committed to re-examine the process for component substitution. NRC Report Para. 7.2, 7.5 (Refer also to Item Nos. 16, 31 and 50)	Developed a new procedure. See status/ comments. Will also clarify definition of design change and clarify process through which a decision to institute a component substitution or design change is made.	Complete*	M.Y. developed a new procedure to provide improved design control and technical evaluation of replacement items. The procedure meets the intent of the EPRI guideline on technical evaluation of replacement items. The procedure provides assurance, as required by ANSI N18.7, that quality at least equivalent to that specified in the original design basis and requirements, material specifications, and inspection requirements are maintained. Developed new definition of design change and new minor modification process.
49	Formal training is not provided either to PED/EAG personnel on the use of ultrasonic flow monitoring equipment or to I&C personnel on maintenance of ultrasonic equipment. NRC Report Para. 7.6, 6.2		Complete	Specific operating and maintenance instructions have been provided with the equipment. Assistance is provided by the manufacturer's representative as required.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
50	The QA Audit Report 88-07 stated that the replacement of batteries 1 & 3 could have been performed as a component substitution instead of as a design change. NRC Report Para. 7.5 (Refer also to Item No. 16)		Complete	This was a gratuitous comment in an audit report. The component substitution process has been re-evaluated. Refer to response to Executive Summary Issue (5).
51A	SCC trip valves design basis correction. (Follow-up to Item No. 4)		Complete	Revised AOP 2-36 to require manual closure of the SCC trip valves within 24 hours of a valid automatic closure.
51B	SCC trip valves design basis correction. (Follow-up to Item No. 4)	Install a drain valve on the bottom of the accumulator tank (TK-110).	1993 Refueling Outage*	Install a drain valve on the bottom of the accumulator tank (TK-110).

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
52	M.Y. audit program (QA) did not identify the programmatic findings identified by the NRC SSFI team. NRC Report Para. 1.0(1)		Complete	Refer to response to Executive Summary Issue (1).
53	The wrong discharge rate was specified for the acceptance test for batteries 2 & 4. The NRC considers this a failure of QC to verify correct test requirements. NRC Report Para. 3.2.4.1(c) (Refer also to Item No. 17)	Incorporate sample verification of the basis of post-modification functional testing acceptance criteria into QA evaluation guidelines and appropriate audit plans.	Complete	A new element was initiated in the QC review to verify the correct test requirements are specified for acceptance tests.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
THE FOLLOWING ITEMS WERE NOT DOCUMENTED IN THE NRC INSPECTION REPORT BUT ARE LISTED HERE AS THEY ARE BEING TRACKED BY MAINE YANKEE AS A FOLLOW-UP TO THE SSFI				
54	FSAR lists all CCW heat loads on Pages 9-39 through 9-42. List should be updated for accuracy; accuracy of loads marked "safeguards" should be checked; definition of "safeguards" loads should be clarified; classification of control room HVAC should be verified.	Performing review.	Complete	Review complete. To be addressed in next FSAR Update.
55	Flow diagram is not clearly labeled to indicate that PCC-S-328 is a vacuum breaker (instead of a safety valve).		Complete	Drawing Change Request (DCR) #89-013 revised the flow diagram.
56	The IASD and Ops Procedure 1-15-1 are inconsistent for the PCC/SCC pH range.		Complete	Error in compiling the IASD. Revised the IASD.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
57	The IASD and the I&C PM sheets are inconsistent for SCC heat exchanger outlet temperature.		Complete	Error in compiling the IASD. Revised the IASD.
58	Correlation between control room temperature and internal instrument cabinet temperature has not been documented.	Measure temperature in the RPS and SPDS cabinets (located in the control room) during functional testing of HELB cabinet instrument modifications.	Complete*	EDCR 89-600 replaced 12 outmoded sigma and heat generating indicators and/or controllers with comparable non-stepping indicators.
59	An NRC commitment included in a CMS was changed without notifying Licensing.		Complete	CMS Procedure 20-3-1 was revised to require notification of Licensing if an NRC commitment is changed.
60	Deficiencies in Surveillance Procedure 3.1.20 including stroke time limit and acceptance criteria. (Refer to Item Nos. 4 and 61)		Complete	Changes to improve Surveillance Procedure 3.1.20, including stroke time limit and acceptance criteria, were provided.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
61	Improvements to Surveillance Procedure 3.1.20. (Refer to Item No. 60)	Revise Surveillance Procedure 3.1.20 per recommendation resulting from Item 60.	Complete*	Procedure revised.
61A	Incorrect pressure for operability stated in Surv. Proc. 3.1.20	Revise Proc. Section 5.4.15	Complete*	Use acceptance criteria in MYP-305, 306. See new Procedure No. 3.1.20-4, Rev. 0.
62	Loss of safeguards pump seal leakage coolers could affect off site doses. See MY 89-934 (Refer to Item No. 54)	Revise the dose calculation to include loss of HPSI, LPSI and CS seal leakage coolers.	Complete*	
63	YNSD recommendation included in MYP 89-480 (Refer to Item No. 54)	Upgrade AC-1A, 1B to S.C.3	09-30-91*	Upgrade AC-1A, 1B to Safety Class 3.
64	SWEC heat balance report recommendation Refer to Item No. 2)	Revise CCW piping to D-G to increase flow.	Complete*	Additional SWEC calc. show flow is adequate. New valve trim installed per EDCR 89-65; flow increase about 40 gpm.

ITEM NUMBER	DESCRIPTION	PLANS	SCHEDULED COMPLETION	STATUS/COMMENTS
65	Spurious TK-110 low pressure alarms when trying to operate at 92 psig. (Refer to Item No. 5/C)	Develop CPA to prevent spurious alarms.	Complete*	YNSD to resize accumulator tank. MYP-90-0481 authorized plant to reduce alarm set pressure to 89 psig.
66	Incorrect acceptance criteria Ops Surv. Proc. 3.1.2.8 and 3.1.2.7 (PCC-72 & SCC-4) IST related. (Refer to Item No. 6)	Develop correct criteria and revise procedures.	Complete*	