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June 7, 1991  
ND3MNO:3142

Beaver Valley Power Station, Unit No. 1  
Docket No. 50-334, License No. DPR-66  
LER 91-013-00

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 91-013-00, 10 CFR 50.73.a.2.i.B, "Pressurizer Code Safety Valve Lift Setting Less Than Technical Specification Limit".

Very truly yours,

T. P. Noonan  
General Manager  
Nuclear Operations

JGT/sl

Attachment

*Handwritten initials/signature*

June 7, 1991

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ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST. SO HAS FORWARDED COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F50), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (1510-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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Pressurizer Code Safety Valve Lift Setting Less Than Technical Specification Limit

EVENT DATE (B)			LER NUMBER (B)			REPORT DATE (D)			OTHER FACILITIES INVOLVED (B)							
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)		
N/A														0   5   0   0   0		
0   5	1   0	9   1	9   1	--	0   1   3	--	0   0	0   6	0   7	9   1				0   5   0   0   0		

OPERATING MODE (B)		6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 6 (Check one or more of the following) (1)				
POWER LEVEL (10)		0 0 0	20.402(b)		20.405(c)	60.73(a)(2)(iv)	73.73(b)
			20.406(a)(1)(i)		60.36(c)(1)	60.73(a)(2)(v)	73.73(c)
			20.406(a)(1)(ii)		60.36(c)(2)	60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)
			20.406(a)(1)(iii)	X	60.73(a)(2)(i)	60.73(a)(2)(vii)(A)	
			20.406(a)(1)(iv)		60.73(a)(2)(ii)	60.73(a)(2)(viii)(B)	
			20.406(a)(1)(v)		60.73(a)(2)(iii)	60.73(a)(2)(ix)	

LISEE CONTACT FOR CHILDER (2)

NAME	TELEPHONE NUMBER
T.P. Noonan, General Manager Nuclear Operations	<div data-bbox="1122 777 1224 779">AREA CODE</div> <div data-bbox="1224 777 1531 779">4 1 2 6 4 3 1 1 2 5 8</div>

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	A B R V	T O 2 0	Y								

SUPPLEMENTAL REPORT EXPECTED (1A)

YES (if yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)			
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**ABSTRACT** (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (58)

On 5/2/91, with the Unit in Refueling, the "C" Pressurizer Code Safety Valve (RV-RC-551C) was tested by Wyle Laboratories in accordance with Beaver Valley Test Procedure (BVT) 1.60.05 "IST - Safety and Relief Valve Tracking." In two of three lift tests, the valve actuated at pressures outside the Technical Specification lift setpoint of 2485 psig  $\pm 1/-3$  percent (2509.8 psig/2410.5 psig). The "as-found" setpoints were: 2431, 2402 and 2373 psig. As a result of the lift test on RV-RC-551C, the "A" Pressurizer Code Safety Valve (RV-RC-551A) was tested on 5/10/91. In one of the three lift tests, this valve also actuated at a pressure outside the lift setpoint tolerance of  $\pm 1/-3$  percent. The "as-found" setpoints were: 2409, 2429 and 2425 psig. The valves were left with Wyle Laboratories for repair and resetting of valve lift pressure to 2485 psig  $\pm 1$  percent. The apparent cause for the failure of the valves to meet the lift setpoint on three successive lifts was setpoint drift. The valves were reset to within  $\pm 1$  percent of the lift setpoint. There were no safety implications as a result of this event. The lift point for RV-RC-551C was found within specification on the first and conservatively low on the second and third lifts. The lift point on RV-RC-551A was found conservatively low on the first lift test and the two subsequent lifts were within specifications.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (PS30), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Beaver Valley Power Station Unit 1	DOCKET NUMBER (2)  0 5 0 0 0 3 3 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION

On 5/2/91, with the Unit in Refueling (Operating Mode 6), the "C" Pressurizer Code Safety Valve (RV-RC-551C) was tested by Wyle Laboratories in accordance with Beaver Valley Test Procedure (BVT) 1.60.05 "IST - Safety and Relief Valve Tracking." In two of three lift tests, the valve actuated at pressures outside the Technical Specification lift setpoint tolerance of 2485 psig  $\pm 1/-3$  percent (2509.8 psig/2410.5 psig). The "as-found" setpoints were: 2431, 2402 and 2373 psig. As a result of the lift test on RV-RC-551C, the "A" Pressurizer Code Safety Valve (RV-RC-551A) was tested on 5/10/91. In one of three lift tests, this valve also actuated at a pressure outside the lift setpoint tolerance of  $\pm 1/-3$  percent. The "as-found" setpoints were: 2409, 2429 and 2425 psig. The valves were left with Wyle Laboratories for repair and resetting of valve lift pressure to 2485 psig  $\pm 1$  percent. The "B" Pressurizer Code Safety Valve (RV-RC-551B) was also tested during this outage. This valve passed its lift setpoint test.

CAUSE OF THE EVENT

The failure of the valves to lift within the setpoint on three successive lifts was attributed to setpoint drift. The valves are Target Rock Pilot Operated Relief/Safety Valves, Model 69C, and are not subject to potential lift setpoint shift problems experienced in the industry resulting from operating the valves in an environment different from that used to establish the lift setpoint.

CORRECTIVE ACTIONS

1. Wyle Laboratories was contracted to repair the valves and verify valve lift pressure setpoint of 2485 psig  $\pm 1$  percent, on three successive tests.
2. The Beaver Valley Test Procedure acceptance criteria is being reevaluated. Potential changes may take credit for only the first ("as-found") lift, since this may be more indicative of actual valve performance.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20545; AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

YEAR

SEQUENTIAL  
NUMBERREVISION  
NUMBER

Beaver Valley Power Station Unit 1

0 5 0 0 0 3 3 4 9 1 — 0 1 3 — 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PREVIOUS OCCURRENCES

There were two previously issued reports on these valves at Beaver Valley. They are Unit 1 LER 84-019-00 "Pressurizer Code Safety Valve (RV-RC-551B) Lift Setting Greater Than Technical Specifications Limit", and Unit 2 LER 89-008-00 "Pressurizer Code Safety Valve (2RCS-RV551B) Lift Setting Greater Than Technical Specifications Limit." The valves in use at Unit 2 are Crosby Valve and Gage Company, Model HB-86-BPE.

SAFETY IMPLICATIONS

There were no safety implications to the public as a result of this event. The valves' actual lift points were found to be conservatively low. As a result of the test results of RV-RC-551A and 551C, the remaining Pressurizer Code Safety Valve (RV-RC-551B) was tested in accordance with ASME Section XI. This redundant code safety valve was found to be within the requirements of Technical Specifications and was considered operational at all times. While the results of RV-RC-551A and 551C show that they would have actuated at a lower pressure, the power operated relief valves (which actuate at 2335 psig) were operable at all times and would have actuated at a pressure still lower than the safety valves did when tested.

REPORTABILITY

This written report is being submitted in accordance with 10CFR50.73.a.2.i.B, as an event or condition which is prohibited by Technical Specifications.