



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 13, 1995

Bucket File

Mr. William J. Cahill, Jr.  
Executive Vice President - Nuclear  
Generation  
Power Authority of the State of  
New York  
123 Main Street  
White Plains, NY 10601

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT, APPROVAL OF ASME CODE CASE  
N-498-1 AS AN ALTERNATIVE TO THE REQUIRED HYDROSTATIC PRESSURE TEST  
(TAC NO. M90801)

Dear Mr. Cahill:

By letter dated November 4, 1994, you submitted two requests for relief from portions of the American Society of Mechanical Engineers (ASME) Code, Section XI, requirements to perform hydrostatic tests of Inservice Inspection (ISI) Class 2 and Class 3 systems at the James A. FitzPatrick Nuclear Power Plant (FitzPatrick). The first part of your request, to use ASME Code Case N-416-1 in lieu of performing the Code-required hydrostatic pressure test for welded repairs and welded pipe replacements, was previously reviewed by the NRC staff and was authorized for testing at FitzPatrick pursuant to Section 50.55a(a)(3)(ii) to Title 10 of the *Code of Federal Regulations* (10 CFR 50.55a(a)(3)(ii)). This authorization, with the attached Safety Evaluation (SE), was transmitted to you by letter dated December 13, 1994.

Your second requested relief, for use of the alternative rules of ASME Section XI Code Case N-498-1, dated May 11, 1994, "Alternative Rules for 10-Year System Hydrostatic Testing for Class 1, 2, and 3 Systems," with the proposed augmented examinations for the 10-year inspection interval hydrostatic testing on the RCIC steam supply piping, and RCIC suction and discharge piping, has been reviewed by the NRC staff.

Based on the information submitted, the proposed alternative is authorized for testing at FitzPatrick pursuant to 10 CFR 50.55a(a)(3)(ii). The proposed alternative is authorized in that compliance with the specified requirements of ASME Code Section XI would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety. Your alternative, use of ASME Code Case N-498-1, is authorized for testing at FitzPatrick until such time as this code case is published in a future revision of Regulatory Guide (RG) 1.147. At that time, if you intend to continue to implement this code case, you are required to follow all provisions in ASME Code Case N-498-1 with limitations issued in RG 1.147, if any.

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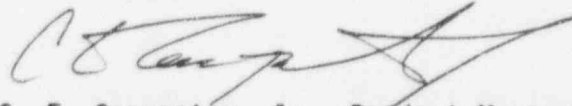
W. Cahill

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A copy of the related Safety Evaluation is enclosed. This completes the NRC staff's review and, by this letter, we are closing TAC No. M90801.

Sincerely,



C. E. Carpenter, Jr., Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure: Safety Evaluation

cc w/encl: See next page

William J. Cahill, Jr.  
Power Authority of the State of New York

James A. FitzPatrick Nuclear  
Power Plant

cc:

Mr. Gerald C. Goldstein  
Assistant General Counsel  
Power Authority of the State  
of New York  
1633 Broadway  
New York, NY 10019

Ms. Donna Ross  
New York State Energy Office  
2 Empire State Plaza  
16th Floor  
Albany, NY 12223

Resident Inspector's Office  
U. S. Nuclear Regulatory Commission  
P.O. Box 136  
Lycoming, NY 13093

Mr. Richard L. Patch, Director  
Quality Assurance  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, NY 10601

Mr. Harry P. Salmon, Jr.  
Resident Manager  
James A. FitzPatrick Nuclear  
Power Plant  
P.O. Box 41  
Lycoming, NY 13093

Ms. Charlene D. Faison  
Director Nuclear Licensing  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, NY 10601

Supervisor  
Town of Scriba  
Route 8, Box 382  
Oswego, NY 13126

Mr. Robert G. Schoenberger,  
First Executive Vice President  
and Chief Operating Officer  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, NY 10601

Charles Donaldson, Esquire  
Assistant Attorney General  
New York Department of Law  
120 Broadway  
New York, NY 10271

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406