

ILLINOIS POWER COMPANY



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P04-83(10-28)-L
500 SOUTH 27TH STREET, DECATUR, ILLINOIS 62525
October 28, 1983

Docket No. 50-461

Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Subject: Clinton Power Station Unit 1
Safety Parameter Display System
Pre-implementation Package

Dear Mr. Schwencer:

NRC Generic Letter 82-33, Supplement 1 to NUREG-0737, provides clarification of NRC requirements related to the Safety Parameter Display System (SPDS), among other Emergency Response Capability initiatives. In a letter, dated April 13, 1983, Illinois Power Company's (IP) Emergency Response Capability Implementation Plan (ERCIP) was provided as required by Generic Letter 82-33. IP Letter U-0647, dated July 5, 1983, provided you staff with the ERCIP schedule that will implement this plan. One of the actions committed to in ERCIP was the submittal of the SPDS Pre-implementation Package by November 1, 1983.

Enclosed is the SPDS Pre-implementation Package. This Package consists of the following documentation:

1. Verification and Validation Plan for Safety Parameter Display System;
2. Safety Parameter Display System Safety Analysis Report;
3. Safety Parameter Display System Parameter Set Validation Report;
4. Human Factors Review of the Safety Parameter Display System; and the
5. SPDS Verification & Validation Team Report on the Human Factors Review of the Safety Parameter Display System.

This documentation relates to the requirements contained in paragraphs 4.1.a, 4.1.b, 4.1.d, 4.1.e, 4.2.a and 4.2.b of NUREG-0737 Supplement #1. In addition, paragraph 4.2.b of

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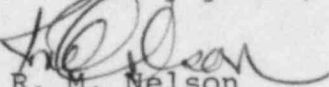
NUREG-0737 Supplement #1 requires that a review of the SPDS (parameters) be conducted in accordance with the CPS Technical Specifications to ensure there are no unreviewed safety questions pertaining to SPDS or its use prior to implementation. As recognized by your Staff in subsection 7.5.2.3 of the CPS Safety Evaluation Report (NUREG-0853), the SPDS is not Class 1E, single failure proof, nor designed to function during seismic events. This is acceptable per paragraph 4.1.c of NUREG-0737 Supplement #1. Based upon this fact, it is the IP position that the SPDS is not required to fulfill any requirements of the CPS Technical Specifications.

It should be noted that the SPDS Safety Analysis Report contains information regarding the presently proposed SPDS parameter set. As a result of an independent verification and validation review of the CPS SPDS parameter set (see attached SPDS Parameter Set Validation Report) some recommended changes to this set have been proposed. These recommendations are being evaluated and IP will provide the revised SPDS Safety Analysis Report in February 1984.

In addition, several recommendations are contained within the two reports regarding human factors aspects of the CPS SPDS (see Items 4 & 5 above). These recommendations are being evaluated and IP will provide the results of our review of these recommendations in February 1984.

Illinois Power representatives would be please to meet with your Staff concerning any aspect of this Preimplementation Package.

Sincerely yours,



R. M. Nelson
Director- Nuclear Licensing
and Configuration
Nuclear Station Engineering

TLR/lgm

Attachments

cc: G. A. Harrison, NRC Clinton Licensing Project Manager
NRC Resident Office
Illinois Department of Nuclear Safety
R. A. Kendall, NRC IC&B
J. Joyce, NRC IC&B