(7-77)

NRC FORM 366

LICENSEE EVENT REPORT

U.S. NUCLEAR REGULATORY COMMISSION
Update Report:
Previous Report Date: 6-28-83

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
0 1	LICENSEE CODE (2)	N C B E P 2
	LICENSE NUMBER (3)	0 0 - 0 0 0 0 0 - 0 0
	LICENSE TYPE (4)	4 1 1 1 1
	CAT (5)	
CON'T 0 1	REPORT SOURCE (6)	L
	DOCKET NUMBER (7)	0 5 0 - 0 3 2 4
	EVENT DATE (8)	0 5 3 0 8 3
	REPORT DATE (9)	1 0 1 7 8 3

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During unit power operation, a suppression pool level high/low alarm annunciation
0 3 occurred. A check of RTGB suppression pool level indicators 2-CAC-LI-2601-1 and
0 4 2601-3 showed a level indication of -18 inches while the remote shutdown panel sup-
0 5 pression pool level indicator, 2-CAC-LI-3342, showed a level indication of -29 inches.
0 6 At the time of this event, the redundant RTGB indicator/recorder, 2-CAC-LR-2602, was
0 7 inoperable to allow the installation of a plant modification. This event did not
0 8 affect the health and safety of the public. Technical Specifications 3.3.5.3,
6.9.1.9b

Line	SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
0 9	I D	D	Z	P I P E X X	A	Z

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 5 4	/	0 3	L	—	1

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
X	Z	Z	Z	0 0 0 0	Y	N	Z	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Demineralized water from a ruptured tygon tube, which was not isolated following a
1 1 HPCI System PT performed on May 29, 1983, was spraying on the 2601-1 and 2601-3 indi-
1 2 cators'.respective transmitter 2-CAC-LT-2601, thereby, causing the indicators to show
1 3 erroneous indications. The subject tygon tube water source was isolated and indica-
1 4 tions shown by the indicators returned to normal. The HPCI PT was revised to ensure
the water supply to the subject tygon tube is isolated following use.

Line	FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
1 5	E	0 9 3	NA	A	Operational Event

Line	ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z	Z	NA	NA

PERSONNEL EXPOSURES

Line	NUMBER (37)	TYPE (38)	DESCRIPTION (39)
1 7	0 0 0	Z	NA

PERSONNEL INJURIES

Line	NUMBER (40)	DESCRIPTION (41)
1 8	0 0 0	NA

8310250396 831017
PDR ADOCK 05000324
S PDR

LOSS OF OR DAMAGE TO FACILITY (43)

Line	TYPE (42)	DESCRIPTION
1 9	Z	NA

IE22

PUBLICITY (45)

Line	ISSUED (44)	DESCRIPTION	NRC USE ONLY
2 0	N	NA	

NAME OF PREPARER M. J. Pastva, Jr. PHONE: 919-457-9521

GPO 917-926

LER ATTACHMENT - RO #2-83-54

Facility: BSEP Unit No. 2 Event Date: May 30, 1983

During unit power operation, a tygon tube which was attached to a demineralized water source on the -17 foot elevation of the Unit No. 2 Reactor Building ruptured and began spraying water on 2-CAC-LT-2601, which is the instrument transmitter of suppression pool level instruments 2-CAC-LI-2601-1 and LI-2601-3. As a result of the spraying water, the LT-2601 sent an erroneous input signal to the indicators, causing them to indicate a suppression pool level which was higher than actual (-18" versus an actual level of -29"). Following the discovery of the ruptured tygon tube, the demineralized water source to the tube was isolated and indications shown by the 2601-1 and 2601-3 indicators returned to normal.

An investigation of this event determined that the subject tygon tube had been utilized for the performance of a periodic test on HPCI System suppression pool level switches where a demineralized water source was required to perform the test. Following the completion of this test (PT-03.1.11P) on May 29, 1983, the demineralized water source isolation valve to the tygon tube was left in the open position. The PT procedure did not dictate to close the demineralized water isolation valve following utilization of the demineralized water source.

As a result of this event, PT-03.1.11P, which is the channel functional test of the ECCS suppression chamber high water level instrumentation, and PT-03.1.11PC, which is the channel calibration and functional test of the ECCS suppression chamber high water level instrumentation, were respectively revised on August 10, 1983, and July 20, 1983, to specify that the subject demineralized water source be isolated after use.

CP&L
Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

83 OCT 19 A9:20

October 17, 1983

FILE: B09-13510C
SERIAL: BSEP/83-3409

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
SUPPLEMENT TO LICENSEE EVENT REPORT 2-83-54

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 2, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within thirty (30) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/jo/LETJO3

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

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