

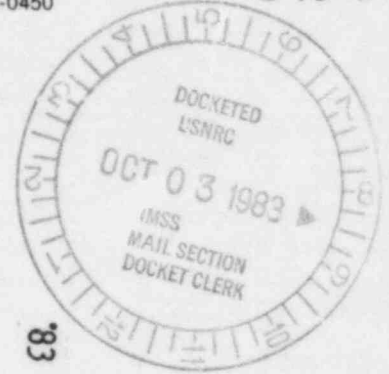


CHEM-NUCLEAR SYSTEMS INC.

240 Stoneridge Drive, Suite 100 • Columbia, South Carolina 29210 • 803/256-0450

71-9105
PDR
Return to
39655

September 30, 1983
RA-0236-3



83 OCT -3 10:55

Mr. Charles E. MacDonald
Chief, Transportation Certification Branch
Division of Fuel Cycle and Material Safety
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr. MacDonald:

REFERENCE: Docket No. 71-9105

We respectfully submit our response (Attachment 1) to the questions raised in your April 25, 1983 letter concerning Model CNS 6-101 shipping package.

Our response is in the form of revised pages to the Safety Analysis Report (Attachment 2). Pursuant to the suggestion in Regulatory Guide 7.9, we have identified revisions by vertical lines in the right margin. We ask that you insert the revised text into the original document sent to you on June 29, 1983.

Please do not hesitate to contact us with any questions regarding this matter.

Sincerely,

L. K. Poppe
Corporate Health Physicist

LKP:jg

Enclosures: Attachment 1 - Response to NRC letter of 4/25/83
Attachment 2 - Revisions to SAR for Model CNS 6-101

ATTACHMENT 1

REQUEST

1. The consolidated application is incomplete in that the information contained in the original application pertaining to the lifting and tie-down lug analysis has not been included.

RESPONSE

This is a self-loading cask which operates in conjunction with the conveyor platform on the trailer deck. Presently, there is no situation expected whereby it would be necessary to remove the cask from the transport vehicle. Therefore, Section 2.4.3 has been revised such that lifting the loaded package is not permitted. Section 2.4.4 has been added and provides an analysis of the cask tie-downs.

REQUEST

2. Derive a maximum heat load for the package design taking into account the definition of low specific activity material [10 CFR 71.4(g)(4)] and the maximum shielding capability of the packaging.

RESPONSE

A 10 watt maximum decay heat load has been specified in Section 1.2.2.2(2). The wattage was derived as requested.

REQUEST

3. Because the secondary containers provide containment, show that there will be no release of radioactive material from the drums as a result of drums crushing against each other during the one-foot free drop of the packaging.

RESPONSE

As the ability to provide this data would require extensive testing and analysis, we have elected to limit the container content to solidified material. This change has been detailed in Section 1.2.2(1).

REQUEST

4. Show that the effectiveness of the internal lead shield plates will not be substantially reduced as a result of the one-foot free drop.

RESPONSE

The internal lead shield plates will no longer be used in conjunction with shipping licensed material. Revised drawings will be forwarded on or before October 14, 1983.

REQUEST

5. The application does not adequately address the areas of acceptance tests and maintenance program.

RESPONSE

A new Chapter 8 has been provided in Attachment 2.

ATTACHMENT 2

INSTRUCTIONS FOR INCORPORATING REVISION 1
CHANGES TO THE CNS 6-101 SAR

REMOVE	INSERT
Title Page	Title Page
Table of Contents	Table of Contents
i	1 and 1a
3	3
4	4, 4a, 4b, and 4c
18	18
21	21
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