



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

January 30, 1995

LTR: BYRON-95-0043
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Mr. John B. Martin
Regional Administrator
U.S. Nuclear Regulatory Commission (Region III)
801 Warrenville Road
Lisle, Illinois 60122-4351

Subject: Byron Station Unit 1
Steam Generator Interim Plugging Criteria 90 Day Report
NPF-37; NRC Docket No. 50-454

- References:
1. October 24, 1994, Letter from George F. Dick (NRR) to D. L. Farrar (ComEd) Issuing Amendment No. 66 to Facility Operating Licenses NPF-37 and 66, Docket Nos. STN 50-454 and STN 50-455.
 2. Draft Generic Letter, "Voltage-Based Repair Criteria For The Repair Of Westinghouse Steam Generator Tubes Affected By Outside Diameter Stress Corrosion Cracking," issued for public comment on August 12, 1994.
 3. October 17, 1994, letter from D. M. Saccomando to Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, transmitting additional information regarding Byron Station IPC submittals and supplements.
 4. September 30, 1994, letter from D. M. Saccomando to W. T. Russell, transmitting additional information regarding Byron Station IPC submittals and supplements.

In Reference 1, NRC approved a license amendment for Byron Station to implement a voltage-based Interim Plugging Criteria (IPC) for Unit 1, Cycle 7. As part of IPC implementation, Technical Specification 4.4.5.5.d.2 requires: "The final results of the inspection and tube integrity evaluation shall be reported to the staff pursuant to Specification 6.9.2 within 90 days following restart."

Pursuant to this requirement, ComEd is submitting the enclosed report of results from the Byron Unit 1 End-of-Cycle (EOC) 6 steam generator inspection. Startup from refueling outage 6 (B1R06) was conducted on November 2, 1994.

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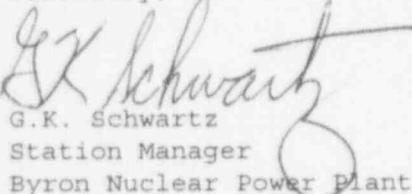
Attachment A to this letter contains the 90 day report for Byron Unit 1. The following information is included in the report, as required by the SER of Reference 1 and the draft Generic Letter (Reference 2):

1. The metallurgical results of the examinations performed on tubes pulled during B1R06.
2. The inspection results in tabular and graphical form for each of the following:
 - a. EOC voltage distribution of all indications regardless of motorized rotating pancake coil (MRPC) confirmation.
 - b. Beginning-of-Cycle (BOC) to EOC cycle growth rate distribution.
 - c. Voltage distribution for EOC repaired indications (from population in item a that were repaired by plugging).
 - d. Voltage distribution for BOC left in service - regardless of MRPC results.
 - e. Voltage distribution for BOC left in service - confirmed by MRPC or not MRPC inspected.
 - f. NDE uncertainly distribution for predicting next EOC voltage distribution.
3. The results of the tube integrity evaluation, including the conditional burst probability and main steam line break leak rate (preliminary results were provided in Reference 3 prior to resumption of power operations from B1R06).

In References 3 and 4, ComEd was required to provide a detailed description of the methods used to evaluate the IPC inspection data. Attachment B contains WCAP-14277, Class 3, "SLB Leak Rate and Tube Burst Probability Analysis Methods for ODS CC at TSP Intersections," which satisfies this commitment. This WCAP is non-proprietary.

Please address any comments or questions regarding this matter to Mr. Jay Smith at (815)234-5441, extension 2604.

Sincerely,


G.K. Schwartz
Station Manager
Byron Nuclear Power Plant

Attachments

cc: Document Control Desk - NRR
G. Dick, Byron Project Manager - NRR
H. Peterson, Senior Resident Inspector - Byron
Office of Nuclear Facility Safety - IDNS

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