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Attached Marked-Up Pages of the Technical Specifications

-		SURVEILLANCE	FREQUENCY
SR	3.3.1.1.16	Verify Turbine Stop Valve Closure and Turbine Control Valve Fast Closure Trip Oil Pressure-Low Functions are not bypassed when THERMAL POWER is ≥ 40% RTP.	18 months
. For Fun 4, and 5 3.3.1.1	3.3.1.1.17 schons 3, in Table -1, the nel sensors excluded.	1. Neutron detectors are excluded. 3. The STAGGERED TEST BASIS Frequency for each Function shall be determined on a per channel basis. Verify the RPS RESPONSE TIME is within limits.	18 months on a STAGGERED TEST BASIS

SURVEILLANCE REQUIREMENTS

- 1. Refer to Table 3.3.5.1-1 to determine which SRs apply for each ECCS Function.
- 2. When a channel is placed in an inoperable status solely for performance of required Surveillances, entry into associated Conditions and Required Actions may be delayed as follows: (a) for up to 6 hours for Functions 3.c, 3.f, 3.g, and 3.h; and (b) for up to 6 hours for Functions other than 3.c, 3.f, 3.g, and 3.h, provided the associated Function or the redundant Function maintains ECCS initiation capability.

		SURVEILLANCE	FREQUENCY
SR	3.3.5.1.1	Perform CHANNEL CHECK.	12 hours
SR	3.3.5.1.2	Perform CHANNEL FUNCTIONAL TEST.	92 days
SR	3.3.5.1.3	Calibrate the analog trip module.	92 days
SR	3.3.5.1.4	Perform CHANNEL CALIBRATION.	18 months
SR	3.3.5.1.5	Perform LOGIC SYSTEM FUNCTIONAL TEST.	18 months
SR <	3.3.5.1.6	The STAGGERED TEST BASIS Frequency shall be determined on a per channel basis for each ECCS trip systems.	7
	Y	Verify the ECCS RESPONSE TIME is within limits.	18 months on a STAGGERED TEST BASIS

Table 3.3.5.1-1 (page 1 of 5)
Emergency Core Cooling System Instrumentation

		FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVETILLANCE REQUIREMENTS	ALLOWABLE VALUE
1.	Inje	Pressure Coolant ection-A (LPCI) and Low ssure Core Spray (LPCS) systems					
		Reactor Vessel Water	1,2,3,	5(p)	8	SR 3.3.5.1.1 SR 3.3.5.1.2	≥ -147.7 inches
		Level - Low Low Low, Level 1	4(e),5(e)			SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5 SR 3.3.5.1.5)
	b.	Drywell Pressure High	1,2,3	2 ^(b)		SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.4	≾ 1.68 psig
	c.	LPCI Pump A	1,2,3,	1	c	SR 3.3.5.1.2	≥ 4.5 second
		Start - Time Delay Logic Card	4(a),5(a)			SR 3.3.5.1.4 SR 3.3.5.1.5	and ≤ 5.5 second
	d.	Reactor Vessel Pressure - Low (Injection Permissive)	1,2,3	4	ç	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 452 psig and ≤ 478 psig
			4(a),5(a)	4	•	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 452 psig and ≤ 478 psig
	e.	LPCS Pump Discharge Flow - Low (Bypass)	1,2,3,	1	Ε	SR 3.3.5.1.1 SR 3.3.5.1.2	≥ 750 gpm
		tow - con (p)pass)	4(2),5(2)			SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	
	f.	LPCI Pump A Discharge	1,2,3,	1	ε	SR 3.3.5.1.1 SR 3.3.5.1.2	≥ 900 gpm
		Flow-Low (Bypass)	4(e),5(e)			SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	
	g.	Manual Initiation	1,2,3,	1	С	SR 3.3.5.1.5	NA
*		1	4(a),5(a)				

⁽a) When associated subsystem(s) are required to be GPERABLE.

⁽b) Also required to initiate the associated diesel generator.

Table 3.3.5.1-1 (page 2 of 5) Emergency Core Cooling System Instrumentation

	FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
	CI B and LPCI C					
	Reactor Vessel Water Level - Low Low Low, Level 1	1,2,3, 4(a),5(a)	2(p)		SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	t -147.7 inches
b.	Drywell Pressure — High	1,2,3	2(b)		SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≤ 1.88 ps(g
c.	LPCI Pump 8 Start - Time Delay Logic Card	1,2,3,	1	c	SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 4.5 second and ≤ 5.5 second
d.	Reactor Vessel Pressure Low (Injection Permissive)	1,2,3	4	•	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 452 psig and ≤ 478 psig
		4(a),5(a)	•	8	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 452 psig and ≤ 478 psig
e.	LPC: Pump 8 and LPC: Pump C Discharge Flow - Low (Bypess)	1,2,3, 4(a),5(a)	1 per pump	ε	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.6	≥ 900 g(xs
f.	Manuel Initiation	1,2,3, 4(a),5(a)	1	С	SR 3.3.5.1.5	KA

⁽a) When associated subsystem(s) are required to be OPERABLE.

⁽b) Also required to initiate the associated diesel generator.

Table 3.3.5.1-1 (page 3 of 5)
Emergency Core Cooling System Instrumentation

	FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REGUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
	gh Pressure Core ray (HPCS) System					
4.	Reactor Vessel Water Level - Low Low, Level 2	1,2,3, 4(a),5(a)	4(b)		SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5 SR 3.3.5.1.5	≥ -47.7 Inches
b.	Drywell Pressure - Kigh	1,2,3	4(p)		SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5 SR 3.3.5.1.5	≤ 1.88 psig
c.	Reactor Vessel Water Level — High, Level B	1,2,3, 4(a),5(a)	2	c	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≤ 54.2 Inches
d.	RCIC Storage Tank Level - Low	1,2,3, 4(c),5(c)	2	0	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 0 Inches
e.	Suppression Pool Water Level - High	1,2,3	2	0	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≤ 12 Inches
1.	HPCS Pump Discharge Pressure - High (Bypess)	1,2,3, 4(a),5(a)	1	E	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 120 psig
9.	Rate - Low (Eypess)	1,2,3, 4(a),5(a)	. 1	Ε	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.3 SR 3.3.5.1.4 SR 3.3.5.1.5	≥ 500 gpm
h.	. Marxual Initiation	1,2,3, 4(a),5(a)	1	c	SR 3.3.5.1.5	KA

⁽a) When associated subsystem(s) are required to be OPERABLE.

⁽b) Also required to initiate the associated diesel generator.

⁽c) When NPCS is OPERABLE for compliance with LCO 3.5.2, "ECCS - Shutdown," and aligned to the RCIC storage tank while tank water level is not within the limits of SR 3.5.2.2.

Primary Containment and Drywell Isolation Instrumentation 3.3.6.1

SURVEILLANCE REQUIREMENTS

- 1. Refer to Table 3.3.6.1-1 to determine which SRs apply for each Primary Containment and Drywell Isolation Function.
- When a channel is placed in an inoperable status solely for performance of required Surveillances, entry into associated Conditions and Required Actions may be delayed for up to 6 hours, provided the associated Function maintains isolation capability.

		SURVEILLANCE	FREQUENCY
SR	3.3.6.1.1	Perform CHANNEL CHECK.	12 hours
SR	3.3.6.1.2	Perform CHANNEL FUNCTIONAL TEST.	92 days
SR	3.3.6.1.3	Calibrate the analog trip module.	92 days
SR	3.3.6.1.4	Perform CHANNEL CALIBRATION.	92 days
SR	3.3.6.1.5	Perform CHANNEL CALIBRATION.	18 months
SR	3.3.6.1.6	Perform LOGIC SYSTEM FUNCTIONAL TEST.	18 months
ha	3.3.6.1.7 nnel sensors excluded.	2.The STAGGERED TEST BASIS Frequency for each Function shall be determined on a per channel basis.	
		Verify the ISOLATION SYSTEM RESPONSE TIME for the main steam isolation valves is within limits.	18 months on a STAGGERED TEST BASIS

		SURVEILLANCE	FREQUENCY
SR	3.5.1.5	Verify each ECCS injection/spray subsystem actuates on an actual or simulated automatic initiation signal.	18 months
SR	3.5.1.6	Valve actuation may be excluded. Verify the ADS actuates on an actual or simulated automatic initiation signal.	18 months
SR	3.5.1.7	Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. Verify each ADS valve opens when manually actuated.	18 months on a STAGGERED TEST BASIS for each valve solenoid
SR	3.5.1.8	ECCS achiation instrumentation is excluded. Verify the ECCS RESPONSE TIME for each ECCS injection/spray subsystem is within limits.	18 months

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Attached Marked-Up
Pages of the Technical Specifications
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SURVEILLANCE REQUIREMENTS (continued)

SR 3.3.1.1.16

This SR ensures that scrams initiated from the Turbine Stop Valve Closure and Turbine Control Valve Fast Closure, Trip Oil Pressure—Low Functions will not be inadvertently bypassed when THERMAL POWER is ≥ 40% RTP. This involves calibration of the Lypass channels. Adequate margins for the instrument setpoint methodology are incorporated into the actual setpoint. Because main turbine bypass flow can affect this setpoint nonconservatively (THERMAL POWER is derived from turbine first stage pressure), the main turbine bypass valves must remain closed at THEPMAL POWER ≥ 40% RTP to ensure that the calibration remains valid.

If any bypass channel setpoint is nonconservative (i.e., the Functions are bypassed at ≥ 40% RTP, either due to open main turbine bypass valve(s) or other reasons), then the affected Turbine Stop Valve Closure and Turbine Control Valve Fast Closure, Trip Oil Pressure—Low Functions are considered inoperable. Alternatively, the bypass channel can be placed in the conservative condition (nonbypass). If placed in the nonbypass condition, this SR is met and the channel is considered OPERABLE.

The Frequency of 18 months is based on engineering judgment and reliability of the components.

SR 3.3.1.1.17

This SR ensures that the individual channel response times are less than or equal to the maximum values assumed in the accident analysis. The RPS RESPONSE TIME acceptance criteria are included in plant Surveillance procedures.

As noted, neutron detectors are excluded from RPS RESPONSE TIME testing because the principles of detector operation virtually ensure an instantaneous response time.

RPS RESPONSE TIME tests are conducted on an 18 month STAGGERED TEST BASIS. Gove of SR 3.3.1.1.17 requires STAGGERED TEST BASIS Frequency for each Function to be determined separately based on the four channels as specified in Table 3.3.1.1-1. This Frequency is based on the logic interrelationships of the various channels required to produce an RPS scram signal.

(continued)

In addition, for Functions 3,4, and ". the a sociated sensors are not required to be response time tested. For these Functions, response time testing for the remaining components, including the ATMS, is required. This allowance is supported by Reference 10.

SURVEILLANCE REQUIREMENTS

SR 3.3.1.1.17 (continued)

Therefore, staggered testing results in response time verification of these devices every 18 months. This Frequency is consistent with the typical industry refueling cycle and is based upon plant operating experience, which shows that random failures of instrumentation components causing serious time degradation, but not channel failure, are infrequent.

REFERENCES

- 1. USAR, Section 7.2.
- 2. USAR, Section 5.2.2.
- USAR, Section 6.3.3.
- 4. USAR, Chapter 15.
- 5. USAR, Section 15.4.1.2.
- NEDO-23842, "Continuous Control Rod Withdrawal in the Startup Range," April 18, 1978.
- 7. USAR, Section 15.4.9.
- Letter, P. Check (NRC) to G. Lainas (NRC), "BWR Scram Discharge System Safety Evaluation," December 1, 1980, as attached to NRC Generic Letter dated December 9, 1980.
- NEDO-30851-P-*, "Technical Specification Improvement Analyses for BWR Reactor Protection System," March 1988.
- 10. NEDO-32291-A, "System Araisses for Elimination of Selected Response Time Testing Requirements,"

 January 1994.

SURVEILLANCE REQUIREMENTS

SR 3.3.5.1.5 (continued)

Frequency. The frequencies recommended by the manufacturer are based on mean time between failure analysis for the components in the associated circuits.

The 18 month Frequency is based on the need to perform this Surveillance under the conditions that apply during a plant outage and the potential for unplanned transients if the Surveillance were performed with the reactor at power. Operating experience has shown these components usually pass the Surveillance when performed at the 18 month Frequency.

SR 3.3.5.1.6

This SR ensures that the individual channel response times are less than or equal to the maximum values assumed in the accident analysis. Response time testing acceptance criteria are included in Reference 5.

ECCS RESPONSE TIME tests are conducted on an 18 month STAGGERED TEST BASIS. This Frequency is consistent with the typical industry refueling cycle and is based upon plant operating experience, which shows that random failures of instrumentation components causing serious response time degradation, but not channel failure, are infrequent.

This SR is modified by a Note that requires the STAGGERED TEST BASIS Frequency for each Function in each trip system to be determined separately based on the number of redundant channels for that Function specified on Table 3.3.5.1-1. This Frequency is based on the logic interrelationships of the various channels required to produce an ECCS initiation signal.

REFERENCES

- USAR, Section 5.2.2.
- 2. USAR, Section 6.3.
- 3. USAR, Chapter 15.

Attachment 4 to U-602376 LS-94-003 Page 5 of 10 ECCS Instrumentation B 3.3.5.1

BASES

REFERENCES (continued) NEDC-30936-P-A, "BWR Owners' Group Technical Specification Improvement Analyses for ECCS Actuation Instrumentation, Part 2," December 1988.

5. / USAR, Section 6.3, Table 6.3-8.

Primary Containment and Drywell Isolation Instrumentation B 3.3.6.1

BASES

SURVEILLANCE
REQUIREMENTS

SR 3.3.6.1.4 and SR 3.3.6.1.5 (continued)

adjusted to account for instrument drifts between successive calibrations consistent with the plant specific setpoint methodology.

The Frequency of SR 3.3.6.1.4 and SR 3.3.6.1.6 is based on the assumption of the magnitude of equipment drift in the setpoint analysis.

SR 3.3.6.1.6

The LOGIC SYSTEM FUNCTIONAL TEST demonstrates the OPERABILITY of the required isolation logic for a specific channel. The system functional testing performed on PCIVs in LCO 3.6.1.3 and on drywell isolation valves in LCO 3.6.5.3 overlaps this Surveillance to provide complete testing of the assumed safety function. The 18 month Frequency is based on the need to perform this Surveillance under the conditions that apply during a plant outage and the potential for an unplanned transient if the Surveillance were performed with the reactor at power. Operating experience has shown these components usually pass the Surveillance when performed at the 18 month Frequency.

The Self Test System may be utilized to perform this testing for those components that it is designed to monitor. Those portions of the solid-state logic not monitored by the Self Test System may be tested at the frequency recommended by the manufacturer, rather than at the specified 18-month Frequency. The frequencies recommended by the manufacturer are based on mean time between failure analysis for the components in the associated circuits.

SR 3.3.6.1.7

This SR ensures that the individual channel response times are less than or equal to the maximum values assumed in the accident analysis. Testing is performed only on channels where the assumed response time does not correspond to the diesel generator (DG) start time. For channels assumed to respond within the DG start time, sufficient margin exists in the 12 second start time when compared to the typical channel response time (milliseconds) so as to assure adequate response without a specific measurement test. The

Primary Containment and Drywell Isolation Instrumentation B 3.3.6.1

BASES

SURVEILLANCE REQUIREMENTS

As noted, the ossociated sensors are not required to be response time tested. Tesponse time tested. Tesponse time testing for the remaining channel components including the ATMS, is required. This is supported by Reference 7.

SR 3.3.6.1.7 (continued)

instrument response times must be added to the MSIV closure times to obtain the ISOLATION SYSTEM RESPONSE TIME. ISOLATION SYSTEM RESPONSE TIME acceptance criteria are included in applicable plant procedures.

The Note to SR 3.3.6.1.7 requires the STAGGERED TEST BASIS Frequency for each Function to be determined seperately based on the number of channels as specified on Table 3.3.6.1-1. This Frequency is based on the logic interrelationships of the various channels required to produce an isolation signal.

ISOLATION SYSTEM RESPONSE TIME tests are conducted on an 18 month STAGGERED TEST BASIS. This Frequency is consistent with the typical industry refueling cycle and is based upon plant operating experience that shows that random failures of instrumentation components causing serious response time degradation, but not channel failure, are infrequent.

REFERENCES

- USAR, Section 6.2.
- 2. USAR, Chapter 15.
- NEDO-31466, "Technical Specification Screening Criteria Application and Risk Assessment," November 1987.
- 4. USAR, Section 9.3.5.
- NEDC-31677-P-A, "Technical Specification Improvement Analysis for BWR Isolation Actuation Instrumentation," June 1989.
- NEDC-30851-P-A, Supplement 2, "Technical Specifications Improvement Analysis for BWR Isolation Instrumentation Common to RPS and ECCS Instrumentation," March 1989.
- 7. NEDO-32291-A, "System Analyses for Elimination of Selected Response Time Testing Requirements,"
 January 1994.

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SURVEILLANCE REQUIREMENTS

SR 3.5.1.7 (continued)

performing this test because valve OPERABILITY and the setpoints for overpressure protection are verified, per ASME requirements, prior to valve installation. Therefore, this SR is modified by a Note that states the Surveillance is not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. SR 3.5.1.6 and the LOGIC SYSTEM FUNCTIONAL TEST performed in LCO 3.3.5.1 overlap this Surveillance to provide complete testing of the assumed safety function.

The Frequency of 18 months on a STAGGERED TEST BASIS ensures that both solenoids for each ADS valve are alternately tested. The Frequency is based on the need to perform this Surveillance under the conditions that apply just prior to or during a startup from a plant outage and the potential for unplanned transients. Operating experience has shown that these components usually pass the SR when performed at the 18 month Frequency, which is based on the refueling cycle. Therefore, the Frequency was concluded to be acceptable from a reliability standpoint.

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REFERENCES

- USAR, Section 6.3.2.2.3.
- 2. USAR, Section 6.3.2.2.4.
- 3. USAR, Section 6.3.2.2.1.
- USAR, Section 6.3.2.2.2.
- USAR, Section 15.2.8.
- 6. USAR, Section 15.6.4.
- USAR, Section 15.6.5.
- 8. 10 CFR 50, Appendix K.
- 9. USAR, Section 6.3.3.
- 10. 10 CFR 50.46.
- 11. USAR, Section 6.3.3.3.

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INSERT B13

SR 3.5.1.8

This SR ensures that the ECCS RESPONSE TIMES are within limits for each of the ECCS injection and spray subsystems. This SR is modified by a note which identifies that the associated ECCS actuation instrumentation is not required to be response time tested. Response time testing of the remaining subsystem components is required. This is supported by Reference 15. Response time testing acceptance criteria are included in Reference 13.

ECCS RESPONSE TIME tests are conducted every 18 months. The 18 month Frequency is based on the need to perform this Surveillance under the conditions that apply during a plant outage and the potential for an unplanned transient if the Surveillance were performed with the reactor at power. Operating experience has shown that these components usually pass the SR when performed at the 18 month Frequency, which is based on the refueling cycle. Therefore, the Frequency was concluded to be acceptable from a reliability standpoint.

INSERT B 3.5-13

REFERENCES (continued)

- Memorandum from R.L. Baer (NRC) to V. Stello, Jr. (NRC), "Recommended Interim Revisions to LCO's for ECCS Components," December 1, 1975.
- 13. USAR, Table 6.3-2.
- 14. USAR, Section 7.3.1.1.1.4.

(15. NEDO-32291-A, "System Analyses for Elimination of Selected Response Time Testing Requirements,"
January 1794.