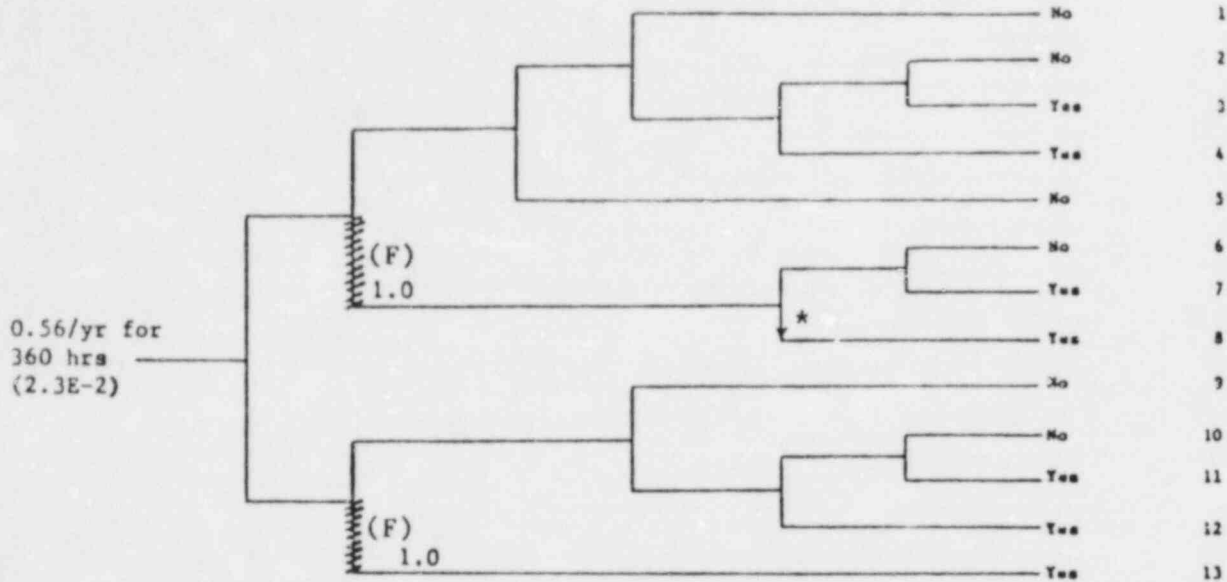


NUREG ASSESSMENT

Loss of Main Feedwater	Reactor Trip	Auxiliary Feedwater and Secondary Heat Removal	PORV Demanded	PORV or PORV Isolation Valve Closure	High Pressure Injection	Long Term Core Cooling	Potential Severe Core Damage	Sequence No.
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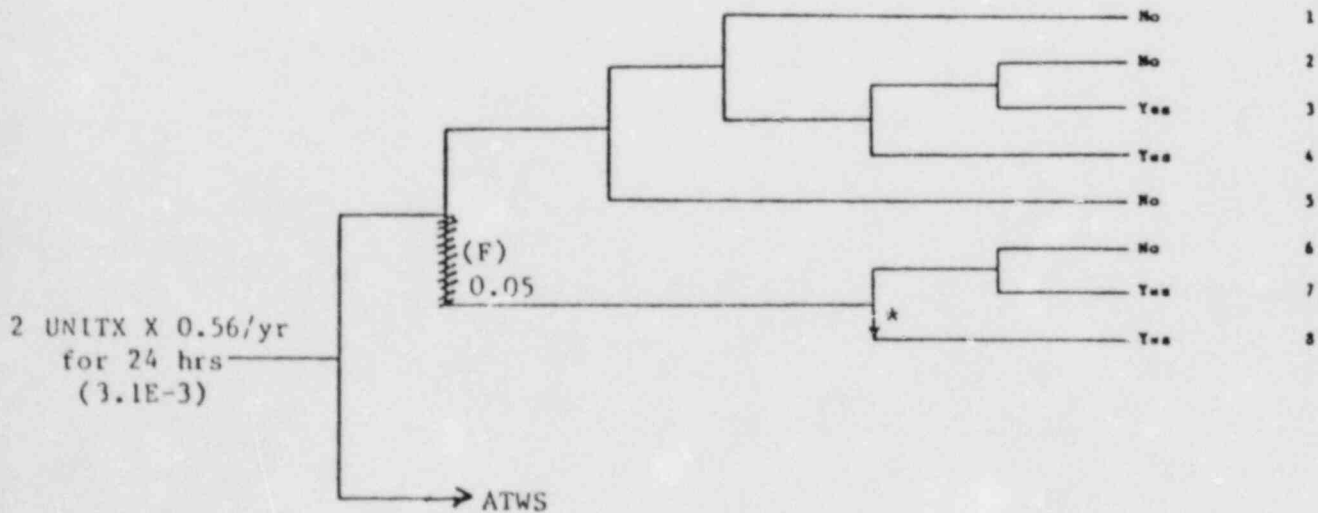
$P = 2.3E-2 + \text{addl. cont. (LOOP), } 1.64E-3 = 2.5E-2 \text{ (SC = 16)}$

NSIC 133704 - Sequence of Interest for Control of Both Auxiliary Feed Pump Turbines Lost at Davis-Besse 1

*not included in mitigation procedures

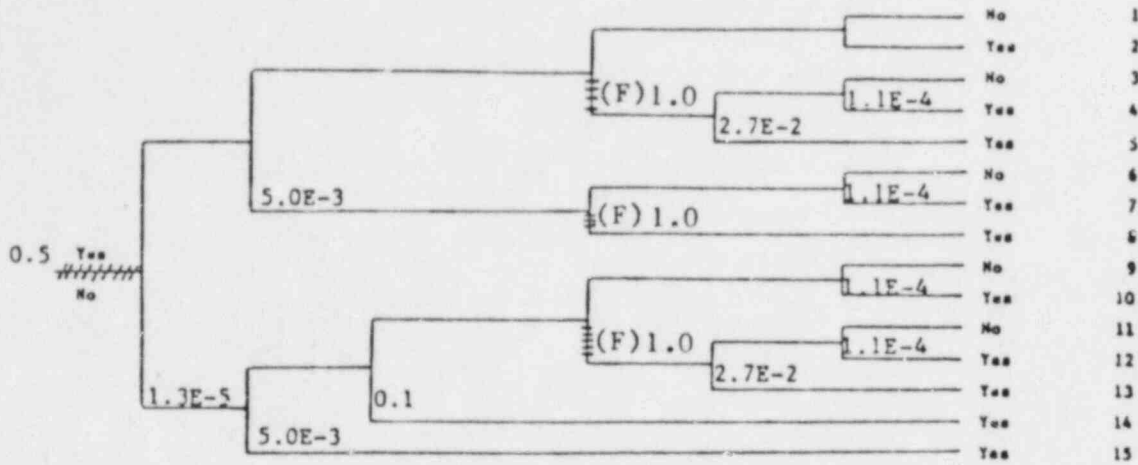
INPO REVISED ASSESSMENT

Loss of Main Feedwater	Reactor Trip	Auxiliary Feedwater and Secondary Heat Removal	PORV Demanded	PORV or PORV Isolation Valve Closure	High Pressure Injection	Long Term Core Cooling	Potential Severe Core Damage	Sequence No.
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$P < 1.55E-4 + \text{addl. cont. (LOOP), } 1.1E-4 < 2.7E-4$

Loss of Offsite Power	Reactor Scram	Diesel Start and Load	Reactor Made Subcritical by the SBLCs Or Rods Are Manually Driven In	RCIC/HPCI ¹ Initiates	ADS/LPCI CS Initiates	Long Term Core Cooling	Potential Severe Core Damage	Sequence No.
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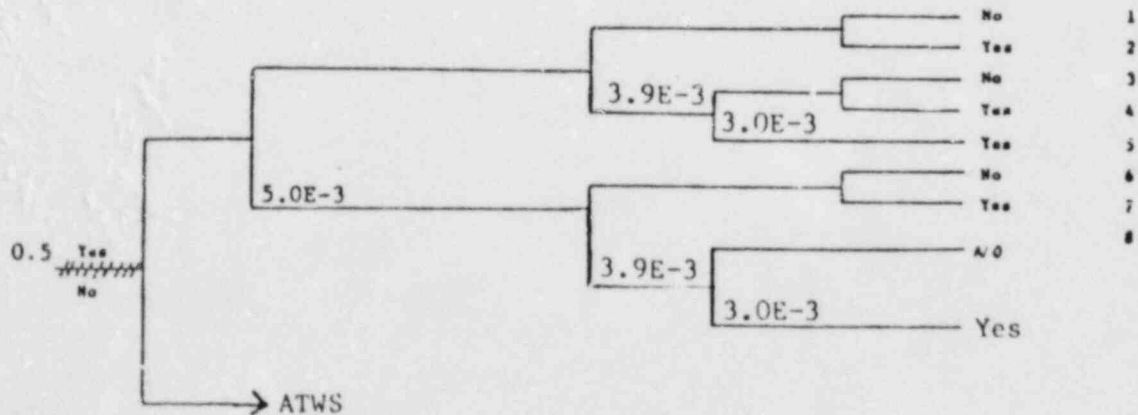
$P = 1.8E-2$ (SC = 18)

NSIC 63129 - Sequence of Interest for a Scram Caused by Electrical Load Rejection at La Crosse

¹La Crosse utilizes shutdown condenser and condensate pumps instead of RCIC and HPCI.

INPO REVISED ASSESSMENT

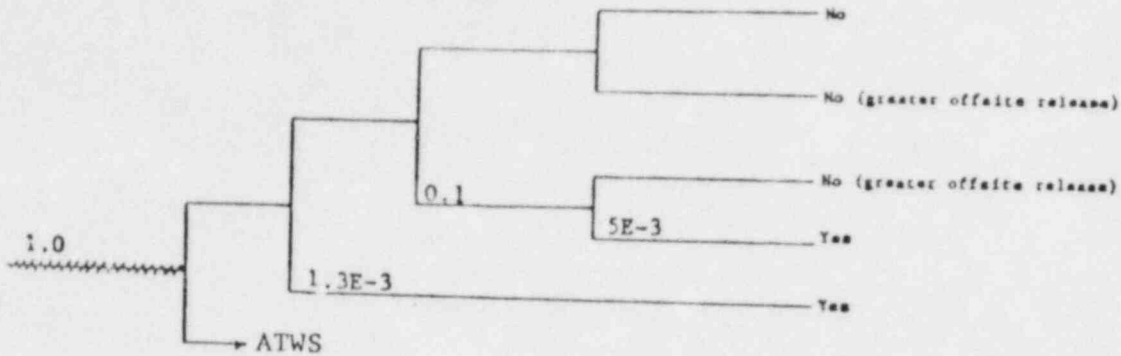
Loss of Offsite Power	Reactor Scram	Diesel Start and Load	Reactor Made Subcritical by the SBLCs Or Rods Are Manually Driven In	RCIC/HPCI ¹ Initiates	ADS/LPCI CS Initiates	Long Term Core Cooling	Potential Severe Core Damage	Sequence No.
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$P < 1.6E-5$

NUREG ASSESSMENT

Steam Generator Tube Rupture	Reactor Trip	Safety Injection	Ruptured Steam Generator Isolated	RUC Depressurized to allow Steam Generator Relief Valve Setpoint	Potential Severe Core Damage
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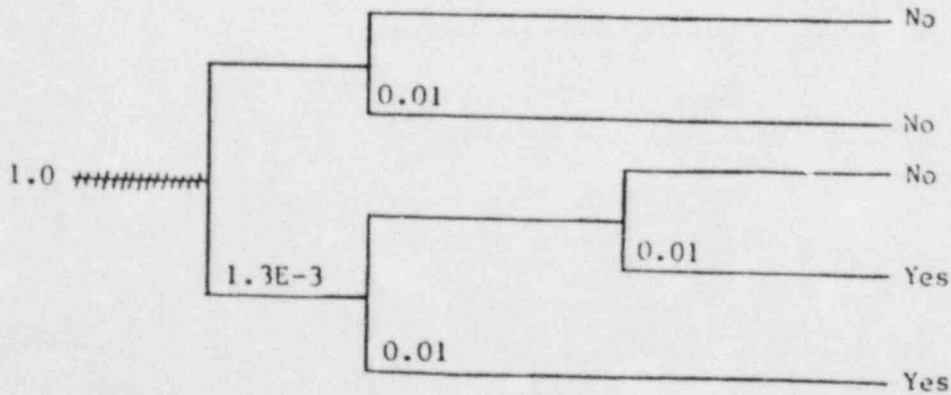


$P = 1.8E-3$ (SC = 27)

MSIC 152563 - Sequence of Interest for Tube Break Occurs at Prairie Island 1

INPO REVISED ASSESSMENT

Steam Generator Tube Rupture	Safety Injection	Ruptured Steam Generator Isolated	Charging Operates and Letdown Isolates	Potential Severe Core Damage
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$P < 2.6E-5$

UNION OF CONCERNED SCIENTISTS

1346 Connecticut Avenue, N.W. • S. 1101 • Washington, DC 20036 • (202) 296-5600

Mr. J. M. Felton, Director
Division of Rules and Records
Office of Administration
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

9 May 1983

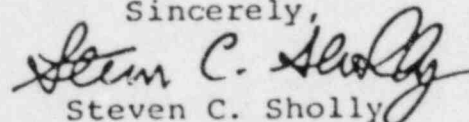
Dear Mr. Felton:

Pursuant to the Freedom of Information Act, please make available at the NRC's Public Document Room at 1717 H Street, N.W., Washington, D.C., copies of records in the following categories:

- A. INPO 82-025, "Review of NRC Report: Precursors to Potential Severe Core Damage Accidents: 1969-1979 A Status Report, NUREG/CR-2497", published by the Institute for Nuclear Power Operations. This report was discussed heavily at an ACRS Subcommittee meeting (Subcommittee on Reliability and Probabilistic Risk Assessment, 3/9/83), and a cover sheet of the report was attached to the slides used in that meeting.
- B. Any and all documents (including correspondence) between and/or among the NRC, the ACRS (including ACRS members, staff, and Fellows), and members of the INPO Review Team which wrote INPO 82-025; these members included W. Pearce, S. Levy, Gareth Parry (NUS), Stan Kaplan (Pickard Lowe and Garrick), Norman Rasmussen, Tony Buhl and Stuart Asselin (Technology for Energy Corporation), and unnamed members of Intermountain Technologies, Inc., regarding the INPO 82-025 study and the NUREG/CR-2497 report.
- C. Any and all reports, memoranda, or any other type of record commenting upon, evaluating, or otherwise discussing INPO 82-025.

Your attention to this request is appreciated. Should you or your staff have any questions regarding this request, please do not hesitate to contact me at the UCS Washington, D.C., office.

Sincerely,


Steven C. Sholly

Technical Research Associate

~~8308220017~~



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JUN 14 1983

Mr. Steven C. Sholly
Technical Research Associate
Union of Concerned Scientists
1346 Connecticut Avenue, N.W.
Washington, DC 20036

IN RESPONSE REFER
TO FOIA-83-242

Dear Mr. Sholly:

This is in partial response to your letter dated May 9, 1983, in which you requested, pursuant to the Freedom of Information Act (FOIA), three separate categories of records which you described in your letter.

Appendix A is a list of documents responsive to items (b) and (c) of your request. These documents are being placed in the NRC Public Document Room (PDR), 1717 H Street, N.W., Washington, DC 20555. Appendix B is a listing of ACRS documents, which have previously been placed in the PDR.

The records requested under item (a) of your request, which are listed below, contain information which the Institute of Nuclear Power Operations (INPO) considers confidential business (proprietary) information. This information is being withheld from public disclosure pursuant to Exemption (4) of the Freedom of Information Act (5 U.S.C. 552(b)(4)) and 10 CFR 9.5(a)(4). We have asked the originators of this information to review their proprietary claim, and will let you know as soon as possible whether the documents, or any portion thereof, may be disclosed to the public.

1. September 9, 1982 - Review of NRC Report: Precursors to Potential Severe Core Damage Accidents: 1969-1979, "A Status Report," NUREG/CR-2497 (INPO 82-025).
2. August 1982 - Notes to INPO on ORNL/NSIC-181/VI prepared by Stan Kaplan of Pickard, Lowe, and Garrick, Inc., (PLG-0243).

Pursuant to 10 CFR 9.9 of the Commission's regulations, it has been determined that the information withheld is exempt from production or disclosure and that its production or disclosure is contrary to the public interest.

~~8308220018~~

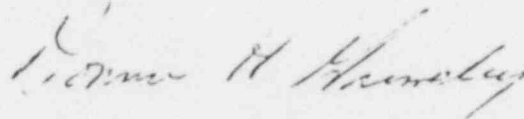
Mr. Steven C. Sholly

-2-

The persons responsible for the denial of document one are the undersigned and Mr. Robert B. Minogue, Director, Office of Nuclear Regulatory Research. Document two is being denied by the undersigned and Mr. Harold R. Denton, Director, Office of Nuclear Reactor Regulation.

These denials may be appealed to the Commission's Executive Director for Operations within 30 days from the receipt of this letter. As provided in 10 CFR 9.11, any such appeal must be in writing, addressed to the Executive Director for Operations, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should clearly state on the envelope and in the letter that it is an "Appeal from an Initial FOIA Decision."

Sincerely,



(for) J. M. Felton, Director
Division of Rules and Records
Office of Administration

Enclosures: As stated