

NRC FORM 366
(12-81)

U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011
EXPIRES 4-30-82

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 M D C C N 1 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 1 7 7 0 8 3 0 8 0 8 0 8 1 9 8 3 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 At 2225, during normal operation, #12 Control Room Air Conditioning
0 3 (AC) unit tripped and would not restart. #12 Control Room AC unit was
0 4 returned to service at 1130 on 8/31/80. #11 Control Room AC unit re-
0 5 mained operable throughout this event per T.S. 3.7.6.1. Unit 2 was oper-
0 6 ating at 100% power.
0 7 Similar event: 50-317/80-39.
0 8

0 9 SYSTEM CODE S G 11 CAUSE CODE B 12 CAUSE SUBCODE A 13 COMPONENT CODE X X X X X X X 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16
17 LER/RO REPORT NUMBER 8 0 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
18 ACTION TAKEN E 19 FUTURE ACTION Z 20 EFFECT ON PLANT Z 21 SHUTDOWN METHOD Z 22 HOURS 0 0 0 0 23 ATTACHMENT SUBMITTED N 24 NPSD-4 FORM SUB. N 25 PRIME COMP. SUPPLIER A 26 COMPONENT MANUFACTURER B 3 5 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The trip was due to high discharge pressure. The unipressure valves were
1 1 adjusted to allow continued operation. To increase the AC system relia-
1 2 bility, the unipressure valves have been replaced with hot-gas bypass
1 3 and pressure regulating valves. No similar AC trips have occurred since
1 4 1980.

1 5 FACILITY STATUS E 28 % POWER 1 0 0 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator Observation 32

1 6 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 AMOUNT OF ACTIVITY Z 34 N/A 35 LOCATION OF RELEASE N/A 36

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43

2 0 PUBLICITY ISSUED N 44 DESCRIPTION N/A 45

NAME OF PREPARER J. S. Lagiewski/R. W. L'Heureux PHONE: 301-269-4492/4503

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PDR ADOCK 05000317
S PDR

NRC USE ONLY

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

August 19, 1983

Dr. Thomas E. Murley
Regional Administrator
U. S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

Docket No. 50-317
License No. DPR 53

Dear Dr. Murley:

Attached is LER 80-49/3X, Revision 1, as required per Technical Specification 6.9.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

LBR Russell

L. B. Russell
Plant Superintendent

LBR:RWL:bsb

cc: Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

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