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September 9, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: LaSalle County Station Units 1 and 2
Monitoring the Feedwater Piping
Response during Pump Trips for Unit 2
NRC Docket Nos. 50-373 and 50-374

Dear Mr. Denton:

The attached revision to FSAR Table 14.2-136 deletes monitoring of the feedwater piping response during pump trips. The justification for deletion of this testing is as follows.

Table 14.2-136 of the LSCS-FSAR states that the feedwater piping response will be monitored during pump trips. This monitoring is being completed for the LaSalle Unit 1 feedwater system. The Unit 1 and 2 feedwater systems are similar, and monitoring feedwater waterhammer responses on Unit 2 can be avoided for the following reasons:

1. The feedwater systems for LaSalle Units 1 and 2 are comprised of the same types of valving and equipment, and the pipe routings are nearly identical. The behavior during pump trips of the Unit 1 feedwater system will therefore be indicative of the Unit 2 system behavior.

2. In addition, the same analytical assumptions concerning the valve and equipment behavior were used for the waterhammer analyses of the feedwater piping in both units. The conservatism of these assumptions will be verified during the testing of Unit 1. Therefore, in light of the above, the response of the Unit 2 feedwater piping during pump trips need not be monitored.

This change was discussed with Mr. Bill Long of NRR on August 15, 1983, and tentatively approved.

This change is expected to be formally included in the next FSAR amendment.

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To the best of my knowledge and belief the statements contained herein and in the enclosure are true and correct. In some respects these statements are not based on my personal knowledge but upon information furnished by other Commonwealth Edison and contractor employees. Such information has been reviewed in accordance with Company practice and I believe it to be reliable.

Enclosed please find one signed original and forty (40) copies of this letter and the enclosure.

If there are any questions in this matter, please contact this office.

Very truly yours,

C W Schroeder 9/12/83

C. W. Schroeder
Nuclear Licensing Administrator

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Enclosure

cc: Region III Inspector - LSCS
Bill Long - NRR

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TABLE 14.2-136

DRYWELL PIPING VIBRATION

PURPOSE

The purpose of this test is to verify that the main steam, reactor recirculation, and feedwater piping have acceptable vibration.

DESCRIPTION

During reactor operation, it is desirable to show that potentially destructive level piping vibrations do not occur by measuring vibration in the recirculation, main steamlines and feedwater system at steady state and during various planned transients.

During operation of the RCIC system all accessible RCIC piping outside containment which is utilized in the flow paths will receive a visual inspection for perceivable vibration under steady-state conditions. The inspection includes branch piping from the main flow paths greater than 2 inches in diameter. If visually perceivable vibration is found, measurements will be taken at the locations of the worst vibration. The measurements will then be compared to the Acceptance Criteria discussed in Table 14.2-16. In addition, the reactor core isolation cooling instrumentation lines on the reactor core isolation cooling steamline outside containment (used to monitor high steam flow and actuate isolation) will be visually inspected to identify any excessive vibration that could result in steady state fatigue failure.

ACCEPTANCE CRITERIA

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The measured displacement for vibration of the recirculation system during recirculation pump trips and subsequent coastdown shall not exceed the allowable values.

The measured range of displacements in the main steamlines during relief valve operation shall not exceed the allowable values.

The measured displacements for steady-state vibration of the recirculation and main steam systems shall not exceed the allowable values.

The measured displacements of main steamlines due to the trip of the turbine stop valve and relief valve operation shall not exceed the allowable values.

TABLE 14.2-136 (Cont'd)

Level 2

The measured vibration displacements of the main steam system following relief valve operation and turbine stop valve trip should not exceed the expected range of displacement.

The measured vibration displacements of the main steam and recirculation systems during steady-state operation should not exceed the expected range of displacement.

Note:

If measured displacements do not meet these criteria, the responsible piping Design Engineer will be contacted for further analysis of the data with regard to design stresses.

The measured vibration stresses induced in the feedwater system following trips of one and both turbine-driven feed pumps and during steady-state operations should not exceed the expected stresses. If measured stresses are greater than expected, further analysis of the data with respect to the design stresses will be performed.

INITIAL CONDITIONS

1. All construction tests and preoperational tests are completed and approved.

2. Instrumentation is installed and calibrated.