



Northeast
Utilities System

107 Selden Street, Berlin, CT 06037

Northeast Utilities Service Company
P.O. Box 270
Hartford, CT 06141-0270
(203) 665-5000

January 5, 1995

Docket No. 50-336
B15057

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 2
Update on the Control Room Emergency Ventilation System

The purpose of this letter is to update Northeast Nuclear Energy Company's (NNECO) letter of June 13, 1994⁽¹⁾ which dealt with the Millstone Unit No. 2 Control Room Emergency Ventilation System (CREVS).

BACKGROUND

On June 2, 1994, during a design review of the Millstone Unit No. 2 Control Room Air Conditioning (CRAC) system, NNECO identified a deficiency in the design of the Control Room Filtration subsystem of the CREVS. A plant design change was implemented that corrected this deficiency. On June 7, 1994, NNECO concluded that the existing assumptions used in the dose calculations for the control room operators were inconsistent with the system design basis. NNECO performed an operability determination (OD) which concluded that based on certain assumptions and the preliminary control room dose estimates, that the CREVS was operable.

As indicated in NNECO's June 13, 1994, letter, the OD is an interim measure to justify operation until a review of proposed plant design and/or design basis changes can be accomplished in accordance with 10CFR50.59. This review is being finalized and is scheduled to be completed prior to startup from the current refueling outage. This letter will update the NRC on the progress that we have made on this task.

(1) J. F. Opeka letter to the U.S. Nuclear Regulatory Commission, "Control Room Emergency Ventilation System," dated June 13, 1994.

A001
/10

DISCUSSION

NNECO has finalized its activities with this task and has been able to revise the radiological calculations such that certain assumptions or actions previously anticipated have been modified, or are no longer necessary.

In NNECO's June 13, 1994, letter, NNECO indicated that one of the assumptions used in the preliminary radiological calculation performed in June 1994 was that "the control room operators will not use self-contained Scott Air Pack breathing apparatus following a main steam line break (MSLB) at Millstone Unit No. 1." NNECO's revised radiological calculations conservatively credit respirator use by the operators from 20 minutes until four hours past the start of the event. In actuality, the equipment that can be used during this event will be either respirators or Scott Air Packs. For conservatism, the revised calculation only credits respirators due to the limitations on their efficiency. The actual equipment that shall be worn, and its required duration, shall be controlled by plant procedures. NNECO's letter of June 13, 1994, also indicated that the preliminary calculations performed in June 1994 do not take credit for purging of the control room. NNECO did indicate in the letter that this issue would be investigated further. NNECO now concludes that for the Millstone Unit No. 1 MSLB case, the control room will be purged at a rate of 1000 cfm. Purging is assumed to begin at 30 minutes after the start of the event.

NNECO indicated in the June 13, 1994, letter that the Millstone Unit No. 1 reactor coolant dose equivalent iodine activity level technical specification value would be lowered from 0.2 $\mu\text{Ci/gm}$ to 0.02 $\mu\text{Ci/gm}$. Although Millstone Unit No. 1 typically operates below 0.02 $\mu\text{Ci/gm}$, the current calculations assume the existing technical specification value. Therefore, it is not necessary to pursue this proposed license amendment.

NNECO will be processing a proposed license amendment for Millstone Unit No. 2 as a result of the revised radiological dose calculations. The proposed license amendment will modify the acceptance criterion for the purification fans from 2500 cfm +/- 10 percent to a new value of 2500 cfm +10 percent, -0 percent. In addition, NNECO will propose to increase the allowed in-leakage rate to the control room from 100 cfm to 130 cfm. This proposed license amendment will be submitted to the NRC by March 31, 1995.

U.S. Nuclear Regulatory Commission
B15057/Page 3
January 5, 1995

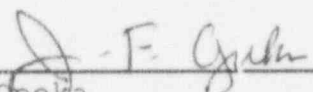
NNECO also indicated in the June 13, 1994, letter that certain design modifications were being considered. NNECO has concluded that the only hardware modification which will be made during the current refuel outage is the installation of an additional register in the control room. This new register will help balance the air flows in the ventilation system. NNECO will also continue to evaluate further potential modifications to the system, as appropriate.

CONCLUSION

The original conclusion developed by NNECO, that the CREVS is operable, continues to remain valid. The complete assumptions and resultant doses used to support NNECO's radiological calculations will be included with the proposed license amendment related to this issue. This letter is for NRC information only. No specific action is requested of the NRC at this time. If you should have any questions, please contact Mr. R. S. Peterson at (203) 440-2074.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY



J. F. Opeka
Executive Vice President

cc: T. T. Martin, Region I Administrator
G. S. Vissing, NRC Project Manager, Millstone Unit No. 2
P. D. Swetland, Senior Resident Inspector, Millstone Unit
Nos. 1, 2, and 3