



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

Docket
File

August 7, 1991

Docket No. 50-334
Serial No. BV-91-29

Mr. J. D. Sieber, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077-0004

Dear Mr. Sieber:

SUBJECT: SAFETY EVALUATION OF THE SECOND TEN-YEAR INTERVAL INSERVICE
INSPECTION PROGRAM PLAN FOR BEAVER VALLEY POWER STATION, UNIT NO. 1
(TAC NO. 65974)

Duquesne Light Company (DLC) submitted the Second Ten-Year Interval Inservice Inspection Program Plan for Beaver Valley Power Station, Unit No. 1 to the NRC by a letter dated August 11, 1989. The NRC requested additional information to complete the review of the inservice inspection (ISI) program on March 1, 1990. The information was provided by a letter dated April 20, 1990, DLC further revised the ISI program by a letter dated June 25, 1990. The Program Plan was submitted for review and evaluation of its compliance with the requirements of the 1983 Edition including the Summer 1983 Addenda of Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, regulations, and plant Technical Specifications.

The NRC staff, with technical assistance from the Idaho National Engineering Laboratory (INEL), has reviewed and evaluated the Program Plan, additional information provided, and requests for relief from some Section XI requirements that DLC determined to be impractical to perform at the facility. Based on information submitted, the NRC staff agrees with INEL's conclusions and recommendations presented in the Technical Evaluation Report enclosed.

The NRC staff has concluded that the Beaver Valley Power Station, Unit No. 1 Second Ten-Year Interval Inservice Inspection Program is in compliance with 10 CFR 50.55a(g). We have determined that Section XI Code requirements for examination of some components cited by DLC are impractical to perform at Beaver Valley Power Station, Unit No. 1 and we have granted relief from those requirements. The NRC staff has determined that granting relief, pursuant to 10 CFR 50.55a(a)(3)(i), (a)(3)(ii) or (g)(6)(i), is authorized by law and will not endanger life, property, or the common defense and security and is otherwise in the public interest giving due consideration to the burden upon DLC that could result if the ASME Code requirements were imposed on the

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Mr. J. D. Sieber

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facility. In cases where the findings could not be made, we have denied the requests. A summary of the requirements and the bases for granting or denying relief requests are contained in the enclosed Safety Evaluation and Technical Evaluation Report.

Sincerely,

/s/

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation
2. Technical Evaluation Report

cc w/enclosures:

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