

Attachment 1  
Millstone Unit No. 3  
Proposed Revision to Technical Specifications  
Reactor Trip System Instrumentation  
Surveillance Requirements Change  
Marked-Up Pages

December 1994

TABLE 4.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. Manual Reactor Trip	N.A.	N.A.	N.A.	R(14, 20)	N.A.	1, 2, 3*, 4*, 5*
2. Power Range, Neutron Flux a. High Setpoint	S	D(2, 4), M(3, 4), Q(4, 6), R(4, 5, 20)	Q	N.A.	N.A.	1, 2
b. Low Setpoint	S	R(4, 20) <sub>51</sub>	S/U(1)	N.A.	N.A.	1***, 2
3. Power Range, Neutron Flux, High Positive Rate	N.A.	R(4, 20) <sub>51</sub>	Q	N.A.	N.A.	1, 2
4. Power Range, Neutron Flux, High Negative Rate	N.A.	R(4, 20) <sub>51</sub>	Q	N.A.	N.A.	1, 2
5. Intermediate Range	S	R(4, 5)	S/U(1)	N.A.	N.A.	1***, 2
6. Source Range, Neutron Flux	S	R(4, 5)	S/U(1), Q(9)	N.A.	N.A.	2**, 3, 4, 5
7. Overtemperature ΔT	S	R	Q	N.A.	N.A.	1, 2
8. Overpower ΔT	S	R	Q	N.A.	N.A.	1, 2
9. Pressurizer Pressure--Low	S	R (20)	Q(18)	N.A.	N.A.	1
10. Pressurizer Pressure--High	S	R (20)	Q(18)	N.A.	N.A.	1, 2
11. Pressurizer Water Level--High	S	R (20)	Q	N.A.	N.A.	1
12. Reactor Coolant Flow--Low	S	R (20)	Q	N.A.	N.A.	

TABLE 4.3-1 (Continued)TABLE NOTATIONS

- \* When the Reactor Trip System breakers are closed and the Control Rod Drive System is capable of rod withdrawal.
  - \*\* Below P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.
  - \*\*\* Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.
  - \*\*\*\* Above the P-9 (Reactor Trip/Turbine Interlock) Setpoint.
- (1) If not performed in previous 31 days.
  - (2) Comparison of calorimetric to excore power indication above 15% of RATED THERMAL POWER. Adjust excore channel gains consistent with calorimetric power if absolute difference is greater than 2%. The provisions of Specification 4.0.4 are not applicable to entry into MODE 2 or 1.
  - (3) Single point comparison of incore to excore AXIAL FLUX DIFFERENCE above 15% of RATED THERMAL POWER. Recalibrate if the absolute difference is greater than or equal to 3%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
  - (4) Neutron detectors may be excluded from CHANNEL CALIBRATION.
  - (5) Detector plateau curves shall be obtained, and evaluated and compared to manufacturer's data. For the <sup>Source Range</sup> Intermediate Range and Power Range Neutron Flux channels the provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
  - (6) Incore - Excore Calibration, above 75% of RATED THERMAL POWER. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
  - (7) Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.
  - (8) (Not used)
  - (9) Quarterly surveillance in MODES 3\*, 4\*, and 5\* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.

Attachment 2  
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TABLE 4.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	TRIP ACTUATING DEVICE OPERATIONAL TEST	ACTUATION LOGIC TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
1. Manual Reactor Trip	N.A.	N.A.	N.A.	R(14,20)	N.A.	1, 2, 3*, 4*, 5*
2. Power Range, Neutron Flux a. High Setpoint	S	D(2, 4), M(3, 4), Q(4, 6), R(4, 5, 20)	Q	N.A.	N.A.	1, 2
b. Low Setpoint	S	R(4, 5, 20)	S/U(1)	N.A.	N.A.	1***, 2
3. Power Range, Neutron Flux, High Positive Rate	N.A.	R(4, 5, 20)	Q	N.A.	N.A.	1, 2
4. Power Range, Neutron Flux, High Negative Rate	N.A.	R(4, 5, 20)	Q	N.A.	N.A.	1, 2
5. Intermediate Range	S	R(4, 5)	S/U(1)	N.A.	N.A.	1***, 2
6. Source Range, Neutron Flux	S	R(4, 5)	S/U(1), Q(9)	N.A.	N.A.	2**, 3, 4, 5
7. Overtemperature $\Delta T$	S	R	Q	N.A.	N.A.	1, 2
8. Overpower $\Delta T$	S	R	Q	N.A.	N.A.	1, 2
9. Pressurizer Pressure--Low	S	R (20)	Q(18)	N.A.	N.A.	1
10. Pressurizer Pressure--High	S	R (20)	Q(18)	N.A.	N.A.	1, 2
11. Pressurizer Water Level--High	S	R (20)	Q	N.A.	N.A.	1
12. Reactor Coolant Flow--Low	S	R (20)	Q	N.A.	N.A.	1

TABLE 4.3-1 (Continued)

TABLE NOTATIONS

- \* When the Reactor Trip System breakers are closed and the Control Rod Drive System is capable of rod withdrawal.
- \*\* Below P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.
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- \*\*\*\* Above the P-9 (Reactor Trip/Turbine Interlock) Setpoint.
- (1) If not performed in previous 31 days.
- (2) Comparison of calorimetric to excore power indication above 15% of RATED THERMAL POWER. Adjust excore channel gains consistent with calorimetric power if absolute difference is greater than 2%. The provisions of Specification 4.0.4 are not applicable to entry into MODE 2 or 1.
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- (6) Incore - Excore Calibration, above 75% of RATED THERMAL POWER. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
- (7) Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (8) (Not used)
- (9) Quarterly surveillance in MODES 3\*, 4\*, and 5\* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.