



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 95 AND 88 TO

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated December 26, 1990, Northern States Power Company (NSP or the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2. The proposed amendments would revise the Technical Specification Tables TS.3.5-4 and TS.4.1-1 to incorporate the feedwater isolation specifications requested by Generic Letter 89-19 and would correct Table TS.4.1-1 to remove surveillances related to the low steam generator water level coincident with steam/feedwater mismatch reactor trip which was removed from the Technical Specifications by License Amendment Numbers 87 and 80, respectively.

2.0 EVALUATION

Generic Letter 89-19, "Request for Action Related to Resolution of Unresolved Safety Issue A-47," recommended that Technical Specifications for all Westinghouse plants include provisions to periodically verify the operability of the main feedwater overflow protection and ensure that the automatic overflow protection is operable during reactor power operation. By letter dated March 15, 1990, the licensee committed to submit a License Amendment Request to revise Technical Specification Table TS.3.5-4 to include limiting conditions for operations for feedwater isolation. The changes to Table TS.3.5-4 are being made in response to that commitment.

Inclusion of feedwater isolation limiting conditions for operations and surveillance requirements for both low and high steam generator level instrumentation will reduce the potential for a steam generator overflow event which may, in turn, reduce the probability of a steamline break or a steam generator tube rupture resulting from an overflow event. As a result, the margin of safety will be enhanced.

The actual surveillance testing performed at power will not be affected by the changes. The changes are only adding currently performed surveillance tests to the Technical Specifications and no additional testing will be required.

9105080142 910430
PDR ADUCK 05000282
P PDR

Based on the above discussions, the staff finds the changes acceptable.

The licensee also proposed to revise Item 12 of Table TS.4.1-1 to eliminate the reference to flow mismatch and to only require surveillance of the steam flow instrument channels and to eliminate the footnote referenced for Item 12.

License Amendment Nos. 87 and 80, respectively, issued by letter dated April 3, 1989, found the Technical Specification changes associated with the elimination of the reactor trip initiated by low steam generator water level coincidence with steam/feedwater mismatch acceptable. However, the limiting conditions for operation and surveillances associated with that reactor trip remained in the Technical Specifications with footnotes stating that they would no longer be applicable following installation of the digital feedwater control system.

Following completion of the installation of the digital feedwater control system, the limiting conditions for operation and associated footnote were removed from the Technical Specifications by License Amendment Nos. 92 and 85, respectively, issued by letter dated March 13, 1990. Due to an oversight, the associated surveillance (Item 12 of Table TS.4.1-1) was not revised to eliminate the reference to flow mismatch and the footnote referenced from Item 12 was not eliminated.

The intent of the proposed changes described above is to complete the removal of the steam/feedwater flow mismatch reactor trip from the Prairie Island Technical Specifications as previously approved by License Amendment Nos. 87 and 80, respectively.

Based on the above discussion, the staff also finds these changes acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the

issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public. The staff therefore concludes that the proposed changes are acceptable.

Principal Contributor: A. Masciantonio

Dated: April 30, 1991