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VPNPD-94-116  
NRC-94-080

October 27, 1994

Document Control Desk  
U.S. NUCLEAR REGULATORY COMMISSION  
Mail Station P1-137  
Washington, DC 20555

Gentlemen:

DOCKETS 50-301  
LICENSEE EVENT REPORT 94-004-00  
LATE REACTOR COOLANT SYSTEM CHLORIDE SAMPLE  
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report 94-004-00 for Point Beach Nuclear Plant, Unit 2. This report is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications." This report describes a reactor coolant system chloride sample that was taken outside the periodicity required by the plant's Technical Specifications.

Please contact us if there are any questions.

Sincerely,

A handwritten signature in cursive script, appearing to read 'R Link', with a small flourish underneath.

Bob Link  
Vice President  
Nuclear Power

TGM/jg

Enclosure

cc NRC Regional Administrator, Region III  
NRC Resident Inspector

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PDR ADDCK 05000301  
S PDR

A subsidiary of Wisconsin Energy Corporation

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## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

POINT BEACH NUCLEAR PLANT, UNIT 2

DOCKET NUMBER (2)

05000301

PAGE (3)

1 OF 3

TITLE (4)

Late Reactor Coolant System Chloride Sample

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	06	94	94	-- 004 --	00	11	06	94	FACILITY NAME	DOCKET NUMBER
										05000
										05000

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)						
POWER LEVEL (10)	0%	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		Abstract below
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		and in Text, NRC Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME

Thomas G. Malanowski, Project Engineer-Licensing

TELEPHONE NUMBER (Include Area Code)

414-221-3950

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE).

X

NO

EXPECTED  
SUBMISSION  
DATE (15)

MONTH

DAY

YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On October 6, 1994, Point Beach Nuclear Plant Unit 2 was in a defueled condition, residual heat removal secured, and the reactor coolant system in reduced inventory. A Unit 2 reactor coolant sample was drawn at 1248 CST to perform a required chloride analysis. Point Beach Nuclear Plant Technical Specifications require a chloride sample three times per week but not to exceed 72 hours between samples when a unit is in refueling shutdown. Due to incomplete communications between chemistry and operations personnel, the reactor coolant system sample was drawn approximately 76.5 hours after the initial sample. Sample results were normal.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)			PAGE (3)
POINT BEACH NUCLEAR PLANT, UNIT 2		05000301		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
				94	-- 004 --	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

## EVENT DESCRIPTION

On October 6, 1994, Point Beach Nuclear Plant (PBNP) Unit 1 was operating at 100% power and Unit 2 was shutdown for its annual refueling outage. Unit 2 was defueled with the reactor coolant system drained to 3/4 pipe, and the residual heat removal system (RHR) secured.

On the morning of October 6, chemistry personnel requested operations to draw a reactor coolant sample for analysis. The sample was requested to meet the requirements of Technical Specification Table 15.4.1-2, "Minimum Frequencies for Equipment and Sampling Tests," Item 1, "Reactor Coolant Samples." This item requires sampling reactor coolant for chlorides three times per week, not to exceed 72 hours between samples during periods of refueling shutdown.

Operations was requested to draw the sample because with RHR secured and the RCS in reduced inventory, normal sample paths through the RHR system are not available. Under normal conditions, with RHR in operation, samples would be drawn by chemistry personnel. Under the existing conditions operations determines an appropriate system sample point to obtain a sample that is best representative of RCS conditions.

In order to meet the Technical Specification requirement, the sample was required to be drawn by 0815 CST on October 6, 1994. When requesting the sample, chemistry personnel did not communicate to operations the requirement to have the sample by this time. The sample was subsequently drawn at 1248 CST, approximately 4.5 hours in excess of the maximum allowed periodicity.

## CAUSES

Chemistry made the appropriate contact with operations to obtain the reactor coolant system sample early on the morning of October 6, 1994. Based on past experience, the request was made in sufficient time to draw the sample in accordance with the Technical Specification requirements. However, chemistry personnel did not communicate the need to obtain the sample by 0815 CST. Since operation personnel are not normally directly involved in obtaining samples this communication oversight resulted in the delay in obtaining the sample.

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			94	-- 004 --	00	

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## CORRECTIVE ACTIONS

The reactor coolant system was sampled. Chloride concentration was verified to be within acceptable limits.

Chemistry procedures were reviewed to ensure that the sample requirements were clearly specified. No deficiencies were noted.

Expectations that any requirements associated with sampling are appropriately communicated to the necessary personnel has been reinforced with the responsible chemistry personnel.

## REPORTABILITY

This event is being reported in accordance with 10 CFR 50.73 (a) (2) (B), "Any operation or condition prohibited by the plant's Technical Specifications."

## SAFETY ASSESSMENT

Reactor coolant system chloride concentration was verified within acceptable limits. As documented in Technical Specification Section 15.3.1, out of specification reactor coolant chemistry is not an immediate safety concern allowing time for corrective action to be taken to restore chemistry within limits. The reactor was defueled and the reactor coolant system was in a condition that is more conservative than that required by Technical Specification 15.3.1.E to correct an out of specification reactor coolant chemistry condition. There are no safety consequences resulting from this event.

## SIMILAR OCCURRENCES:

The following LERs document late or missed surveillances related to inadequate communications:

301/94-002-00 Quarterly Technical Specifications Test of PORV Block Valve not performed.