

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

- CONTROL BLOCK:

						(1)
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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

(1)	N	V	I	P	S	2	(7)	0	0	-	0	0	0	0	-	0	0	(3)	4	1	1	1	1	(4)			(5)
9	8	LICENSEE CODE	14	13	LICENSE NUMBER	25	26	LICENSE TYPE	JU	57	CAT	58															

447

REPORT SOURCE: L 5 0 5 0 0 0 2 4 7 7 0 7 1 6 8 3 9 0 8 / 5 8 3 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

2 On two occasions during normal plant operation, one of three required
3 Boron Injection Tank (BIT) level channels became inoperable (Tech. Spec.
4 3.3.A). The installed spare channel was placed in service. The health
5 and safety of the public were unaffected. Previous similar events:
6 LER-83-024, 83-016, 82-027, 82-023, 82-012.
7
8

9		SYSTEM CODE S F 11		CAUSE CODE B 12		CAUSE SUBCODE A 13		COMPONENT CODE I N S T R U 14				COMP. SUBCODE T 15		VALVE SUBCODE Z 16	
17		EVENT YEAR 8 3 21		SEQUENTIAL REPORT NO. 0 2 8 24		OCCURRENCE CODE 0 3 28		REPORT TYPE L 30		REVISION NO. 0 32					
ACTION TAKEN B 18		FUTURE ACTION F 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER N 25	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27												COMPONENT MANUFACTURER B 0 8 0 0 29			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

0 Solidification of boric acid in the instrument impulse line resulted
1 in an erroneous indication of level. The affected line heat trace
2 temperature setting was increased, the line was cleared and the
3 channel returned to service as an installed spare. A modification of
4 the system is scheduled for the 1984 Refueling Outage.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
5	8	1	0	0	NA	A	Operator Observation		
ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE			
6	2	2	NA	NA					
PERSONNEL EXPOSURES		TYPE		DESCRIPTION					
7	0	0	Z	NA					
PERSONNEL INJURIES		DESCRIPTION							
8	0	0	NA						
LOSS OF INFORMATION TO FACILITY		DESCRIPTION		8309060260 830815 PDR ADOCK 05000247 S PDR					
9	Z	NA	JE22 11						
PUBLICITY		DESCRIPTION		NRC USE ONLY					
10	Z	NA							

Name: Gary Hinrichs

ATTACHMENT

Docket No. 50-247
LER 83-028/03L-0

Consolidated Edison Co. of New York, Inc.
Indian Point Station Unit 2

On July 16, 1983 during normal operation, the Central Control Room operator observed a decrease in the indicated level in one of the three required level channels (LT0994C) for the Boron Injection Tank (BIT), (Technical Specification 3.3.A). The level indication was decreasing due to boric acid solidification near the pressure sensing bellows. The spare level channel was placed in service. The temperature setting of the electric heat tracing for the instrument line associated with LT-994C was increased. The line was cleared and made available as a spare.

On July 21, 1983 the CCR operator observed one of three level channels for the BIT (LT-994B) had failed low. The installed spare (LT-994C) was placed in service.

Due to excessive failures of BIT level indication caused by solidification of boric acid in the instrument impulse line or transmitter bellows, an investigation is underway on feasible ways to modify the heat tracing in order to increase reliability. Level transmitters of an improved design are also being investigated.

These modifications are tentatively planned for the 1984 Refueling/Maintenance Outage.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

August 15, 1983

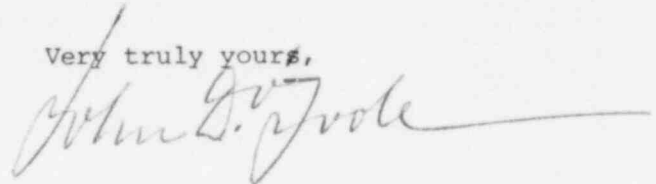
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-83-028/03L-0

Dr. Thomas E. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

Dear Dr. Murley:

The attached Licensee Event Report LER-83-028/03L-0 is hereby submitted in accordance with the requirements of Technical Specification 6.9.1.7. This event is of the type described in Technical Specification 6.9.1.7.2.b.

Very truly yours,



Attach.

CC: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Thomas Foley, Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

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