

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-311
 Unit Name Salem # 2
 Date June 10, 1983
 Telephone 609-935-6000
 Extension 4455

Completed by L. K. Miller

Month May 1983

Day Average Daily Power Level
 (MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>

Day Average Daily Power Level
 (MWe-NET)

16	<u>0</u>
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

IE24
 1/1

OPERATING DATA REPORT

Docket No. 50-311
 Date June 10, 1983
 Telephone 935-6000
 Extension 4455

Completed by L. K. Miller

Operating Status

1. Unit Name	<u>Salem No. 2</u>	<u>Notes</u>
2. Reporting Period	<u>May 1983</u>	
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1162</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason <u>N/A</u>		
9. Power Level to Which Restricted, if any (Net MWe) <u>0</u>		
10. Reasons for Restrictions, if any <u>REFUELING OUTAGE</u>		
	<u>This Month</u>	<u>Year to Date</u>
	<u>Cumulative</u>	
11. Hours in Reporting Period	<u>744</u>	<u>3623</u>
12. No. of Hrs. Reactor was Critical	<u>0</u>	<u>470.3</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>3.3</u>
14. Hours Generator On-Line	<u>0</u>	<u>459.4</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>1109383.2</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>257340</u>
18. Net Elec. Energy Generated (MWH)	<u>(2248)</u>	<u>227030</u>
19. Unit Service Factor	<u>0</u>	<u>12.7</u>
20. Unit Availability Factor	<u>0</u>	<u>12.7</u>
21. Unit Capacity Factor (using MDC Net)	<u>0</u>	<u>5.7</u>
22. Unit Capacity Factor (using DER Net)	<u>0</u>	<u>5.6</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>8.6</u>
24. Shutdowns scheduled over next 6 months (type, date and duration of each) <div style="text-align: center;"><u>N/A</u></div>		
25. If shutdown at end of Report Period, Estimated Date of Startup: <div style="text-align: center;"><u>July 2, 1983</u></div>		
26. Units in Test Status (Prior to Commercial Operation):		
	<u>Forecast</u>	<u>Achieved</u>
Initial Criticality	<u>6/30/80</u>	<u>8/2/80</u>
Initial Electricity	<u>9/1/80</u>	<u>6/3/81</u>
Commercial Operation	<u>9/24/81</u>	<u>10/13/81</u>

8-1-7.R2

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH May 1983

Docket No. 50-311
Unit Name Salem No.2
Date June 10, 1983
Telephone 609-935-6000
Extension 4455

Completed by L.K. Miller

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
3-046	1/21	S	3121.6	C	4	---	HA	GENERA	Nuclear Normal Refueling
83-046	1/21	S	3121.6	C	4	---	HA	TURBIN	Misc Major Condenser Overhaul
83-046	1/21	S	3121.6	C	4	---	HC	HTEXCH	Major Turbine Overhaul Gen Overhaul Items
83-046	1/21	S	3121.6	C	4	---	CA	FUELXX	Generator Major Over- haul (Gen Overhaul Item)

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

4 Exhibit G
Instructions
for Prepara-
tion of Data
Entry Sheets
for Licensee
Event Report
(LER) File
(NUREG 0161)

5 Exhibit 1
Salem as
Source

MAJOR PLANT MODIFICATIONS

REPORT MONTH MAY 1983DOCKET NO: 50-311UNIT NAME: SALEM 2DATE: June 10, 1983COMPLETED BY: L. K. MillerTELEPHONE: 609/935-6000 Ext. 4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-1096	Electrical	Add a fuse in series with primary device on each electrical penetration circuit.
2EC-1104	Control Room Overhead Annunciators (X240)	Replace existing annunciator windows with temporary "Dymo" labels attached with new permanently engraved annunciator windows.
2EC-1457	Heating Water	Reroute heating water piping to alleviate condenser retubing interferences.
2EC-1494	N ₂ , H ₂ , CO ₂ and Turbine Oil Vents ²	Reroute N ₂ , H ₂ , CO ₂ and turbine oil vent piping discharge side of No. 23 Main Condenser shell.
2EC-1570	Turbine Auxiliary Cooling	Replace valves TC-120, 129, 138 and 147 with plug valves of similar construction.
2EC-1599	Pressurizer Overpressure Protection	Perform modifications to valves 2PR47 and 2PR48.
2EC-1609	Reactor Coolant	Install in-line orifice on Barksdale pressure switch in #21, 22, 23 Reactor Coolant Pump Lift Oil Systems. The orifice is to be .008" in diameter.
2SC-0454	Reactor Coolant/Reactor Vessel Head	Install new coupling, Part #C119D31 in place of existing Marman clamps, Part #H 1002306 during reassembly of all five incore instrument conoseal joints in accordance with Station Maintenance Procedure MBC-1.
2SC-0503	Steam Generator Feedwater	Replace 21A&B/22A&B steam generator feed pump turbine lube oil coolers with plate type heat exchangers of alternate material.
2SC-0907	Turbine Generator	Change overspeed trip setting on the Unit 2 turbine generator to 1854 rpm or 3%.
2SC-0933	Main Turbines	Install grounding braids on the shaft of #2 Main Turbine L.P.
2SC-1033	Turbine Low Vacuum Trip and Alarm	Provide a suitable replacement for 2PD-188.
2SC-1063	Structural	Design or fabricate reinforcement to radiation shield beams on El. 151', #2 Unit containment access hatch.

MAJOR PLANT MODIFICATIONS
REPORT MONTH MAY

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: June 10, 1983
COMPLETED BY: L.K. Miller
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2EC-1096	The change provides extra protection in the event of short circuits in the electrical penetrations. No unreviewed safety or environmental questions are involved.	
2EC-1104	This change improves the presentation of alarm information and does not alter any function. No unreviewed safety or environmental questions are involved.	
2EC-1457	This change to the heating system does not affect presently performed safety analysis nor does it create any new hazards. No unreviewed safety or environmental questions are involved.	
2EC-1494	These changes do not affect presently performed safety analysis nor does it create any new hazards. No unreviewed safety or environmental questions are involved.	
2EC-1570	This system does not affect the safe shutdown of the plant, accident mitigation, or radioactivity release, nor does it affect the plant discharge. No unreviewed safety or environmental questions are involved.	
2EC-1599	The valve internals modification is to improve their performance. This modification will not affect any plant process or discharge. No unreviewed safety or environmental questions are involved.	
2EC-1609	This system does not affect the safe shutdown of the plant. The design change has no effect on the plant discharge. No unreviewed safety or environmental questions are involved.	
2SC-0454	This new style coupling has been in service in Unit No. 1 since 1980 with no adverse effects. No unreviewed safety or environmental questions are involved.	
2SC-0503	These replacements will upgrade the heat exchangers with more erosion resistant material. No unreviewed safety or environmental questions are involved.	

* Design Change Request

MAJOR PLANT MODIFICATIONS
REPORT MONTH MAY

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: June 10, 1983
COMPLETED BY: L.K. Miller
TELEPHONE: 609/935-6000 X4455

*DCR NO. 10CFR50.59 SAFETY EVALUATION

- 2SC-0907 This change will raise the turbine overspeed trip setting to allow the unit to operate without unnecessary trips and still remain within the standard Westinghouse recommendations. No unreviewed safety or environmental questions are involved.
- 2SC-0933 This change does not affect the safe shutdown of the plant nor does it alter any plant process or discharge. No unreviewed safety or environmental questions are involved.
- 2SC-1033 There is no change in the function of the switches. This modification will not alter any plant process or discharge. No unreviewed safety or environmental questions are involved.
- 2SC-1063 Provision of a rigid setting surface for the concrete shield blocks in front of the equipment hatch will not affect safe shutdown of the reactor or any plant process or discharge. No unreviewed safety or environmental questions are involved.

P S E & G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT NO. 2

WO NO DEPT UNIT EQUIPMENT IDENTIFICATION

909767 MD 2 22 CC HT EXCHANGER

FAILURE DESCRIPTION: SERVICE WATER IN HEAT EXCHANGER. 021112.

CORRECTIVE ACTION: FIXED BAD TUBE ROW 16 TUBE 47. 821122.

909845 MD 2 22 CHILLER COMPRESS

FAILURE DESCRIPTION: SUPPORT THERMAC IN REPLACEMENT OF 22 CHILLER COMPRESSOR. 821122.

CORRECTIVE ACTION: INSTALLED 22 CHILLER COMPRESSOR. 821201.

910227 MD 2 VALVE, 23CC293

FAILURE DESCRIPTION: LEAKS BY SEAT. 830104.

CORRECTIVE ACTION: MACHINED BACK SEAT, LAPPED IN BODY SEAT, SKIM CUT DISC, BLUE CHECKED 100X.

922099 MD 2 21 COMP COOLING PUMP

FAILURE DESCRIPTION: EXCESSIVE OIL LEAK ON OUTBOARD PUMP BEARING. 830118.

CORRECTIVE ACTION: REPLACED OUTBOARD OIL SEAL AND GASKET ON BEARING HOUSING. 830323.

922775 MD 2 CONT AIR LOCK EL 130

FAILURE DESCRIPTION: REPAIR AIR LOCK INNER DOOR. 830319.

CORRECTIVE ACTION: REPLACED ROLLER BEARING. 830320.

922777 MD 2 VALVE, 2RH20

FAILURE DESCRIPTION: 2RH20 LEAKING APPROX 1 GPM AT BODY TO BONNET FLANGE. 830220.

CORRECTIVE ACTION: FOUND ALL STUDS LOOSE (HAND TIGHT). SLUGGED UP ON NUTS AND TIGHTENED LOOSE PACKING GLAND. 830324.

P S E & G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT NO. 2

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
92787	MD	2	VALVE, 2SJ4
FAILURE DESCRIPTION: VALVE SEAT SEEMS TO BE LEAKING THROUGH. 830321.			
CORRECTIVE ACTION: RESET TORQUE SWITCH TO 2 AS PER ENG LETTER. 830331.			
922794	MD	2	21 CHARGING PUMP
FAILURE DESCRIPTION: AUX OIL PUMP DOES NOT START WHEN 21 CHARGING PUMP TAKEN OUT OF SERVICE. 830322.			
CORRECTIVE ACTION: PERFORMED M31 ON 2PD-8488 AND 2AD-M-762. 830322.			
922795	MD	2	CONT AIR LOCK EL 100
FAILURE DESCRIPTION: AIR LOCK OUTER DOOR DOES NOT SHUT PROPERLY. 830322.			
CORRECTIVE ACTION: BOLTS ON BEARING HOUSING LOOSE AND SOME WERE OUT. RETIGHTENED AND RETORQUED. 830322.			
922803	MD	2	VALVE, 22SW20
FAILURE DESCRIPTION: VALVE WILL NOT OPERATE ELECTRICALLY. 830323.			
CORRECTIVE ACTION: REPLACED MOTOR OPERATOR AND HEATER OVERLOADS. TESTED SAT. 830324.			
922815	MD	2	CONT AIR LOCK EL 100
FAILURE DESCRIPTION: OUTER DOOR NOT CLOSING RIGHT. 830327.			
CORRECTIVE ACTION: REPLACED BROKEN BOLT ON HAND WHEEL. 830328.			
924696	MD	2	FUEL TRANSFER SYSTEM
FAILURE DESCRIPTION: UPENDER DOES NOT OPERATE PROPERLY. 830321.			
CORRECTIVE ACTION: UPENDER FAILED OPEN ON THE BRAKE SOLENOID. REPLACED SOLENOID WITH ONE FROM UNIT 1. 830322.			

P S E & G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT NO. 2

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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924726	MD	2	CONT AIR LOCK EL 130
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FAILURE DESCRIPTION: LATCH ROLLER ON INNER DOOR BROKEN. 830324.

CORRECTIVE ACTION: REPLACED ROLLER AND COTTER PINS. 830324.

993018	MD	2	21 STEAM GENERATOR
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FAILURE DESCRIPTION: PERFORM EDDY CURRENT EXAM IAW T/S 4.6.6.0. 820606.

CORRECTIVE ACTION: 100% OF TUBES WERE EXAMINED IN HOT AND COLD LEGS. NO TUBES PLUGGED. 830325.

993019	MD	2	22 STEAM GENERATOR
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FAILURE DESCRIPTION: PERFORM EDDY CURRENT EXAM IAW T/S 4.4.6.0. 820606.

CORRECTIVE ACTION: 100% OF TUBES WERE EXAMINED IN BOTH HOT AND COLD LEGS. TWO (2) TUBES WERE PLUGGED. 830325.

993020	MD	2	23 STEAM GENERATOR
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FAILURE DESCRIPTION: PERFORM EDDY CURRENT EXAM IAW T/S 4.4.6.0. 820606.

CORRECTIVE ACTION: 100% OF TUBES WERE EXAMINED IN BOTH HOT AND COLD LEGS. NO TUBES WERE PLUGGED. 830325.

993021	MD	2	24 STEAM GENERATOR
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FAILURE DESCRIPTION: PERFORM EDDY CURRENT EXAM IAW T/S 4.4.6.0. 820606.

CORRECTIVE ACTION: 100% OF TUBES WERE EXAMINED IN BOTH HOT AND COLD LEGS. NO TUBES PLUGGED. 830325

SALEM UNIT 2
OPERATIONS SUMMARY REPORT
MAY 1983

The First Refueling Outage of Salem Unit No. 2 commenced on January 21, 1983, and has continued through the month of May.

REFUELING INFORMATION

COMPLETED BY: L.K. MillerDOCKET NO.: 50-311UNIT NAME: Salem 2DATE: June 10, 1983TELEPHONE: 609/935-6000EXTENSION: 4455Month May 1983

1. Refueling information has changed from last month:

YES _____ NO X

2. Scheduled date for next refueling:
- March 31, 1984

3. Scheduled date for restart following refueling:
- June 10, 1984

4. A) Will Technical Specification changes or other license amendments be required?

YES _____ NO XNOT DETERMINED TO DATE 6/1/83

- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES _____ NO XIf no, when is it scheduled? February 1984

5. Scheduled date(s) for submitting proposed licensing action:
-
- March 1984 (if required)

6. Important licensing considerations associated with refueling:
-
- NONE

7. Number of Fuel Assemblies:

A) Incore

193

B) In Spent Fuel Storage

72

8. Present licensed spent fuel storage capacity:

1170

Future spent fuel storage capacity:

1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity:

March 2000

8-1-7.R4



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

June 10, 1983

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

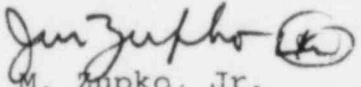
Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of May 1983 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Summary of Safety Related Maintenance
Operating Summary
Refueling Information

Sincerely yours,


J. M. Zupko, Jr.
General Manager - Salem Operations

LKM:sbh

cc: Mr. J. Allan
Acting Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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