NRC FORM 366
(12-81)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011
EXPIRES 4-30-82

CONTROL BLOCK: ☐☐☐☐☐☐ ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: M D C C N 2 ②	LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 ③	LICENSE TYPE: 4 1 1 1 1 ④	CAT ⑤		
CON'T 0 1	REPORT SOURCE: L ⑥	DOCKET NUMBER: 0 5 0 0 0 3 1 8 ⑦	EVENT DATE: 0 7 1 9 8 3 ⑧	REPORT DATE: 0 8 1 7 8 3 ⑨		

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 At 0900 during surveillance test of reactor trip circuit breakers (TCB)
0 3 undervoltage (U/V) device, the response time of TCB 4 and 7 U/V trip de-
0 4 vices was slower than allowed by T.S. 3.3.1.1. The breakers and U/V de-
0 5 vices were adjusted and tested satisfactorily at 1700. TCB 4 and 7 were
0 6 operable as the redundant shunt trip device was operational with satis-
0 7 factory response time.
0 8 Similar events: none.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	I A ⑪	E ⑫	X ⑬	R E L A Y X ⑭	D ⑮	Z ⑯

⑰ LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 3 6	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS ㉒	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER ㉖
E ⑱	X ⑲	Z ⑳	Z ㉑	0 0 0 0	Y ㉓	N ㉔	N ㉕	G 0 8 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 The cause of this event in under investigation. The remaining TCBs were
1 1 similarly response time tested and adjusted as required. Failure trend-
1 2 ing is being conducted under a newly implemented surveillance and pre-
1 3 ventative maintenance program. An update report will be submitted when
1 4 the cause is determined.

	FACILITY STATUS	% POWER	OTHER STATUS ㉚	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION ㉜
1 5	E ㉘	1 0 0 ㉙	N/A	C ㉛	Surveillance Testing

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY ㉟	LOCATION OF RELEASE ㊱
1 6	Z ㉝	Z ㉞	N/A	N/A

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION ㊴
1 7	0 0 0 ㊲	Z ㊳	N/A

PERSONNEL INJURIES

	NUMBER	DESCRIPTION ㊶
1 8	0 0 0 ㊵	N/A

8309060139 830817
PDR ADOCK 05000318
S PDR

LOSS OF OR DAMAGE TO FACILITY ㊸

	TYPE	DESCRIPTION
1 9	Z ㊷	N/A

PUBLICITY

	ISSUED	DESCRIPTION ㊺	NRC USE ONLY
2 0	N ㊹	N/A	

IE22

NAME OF PREPARER M. A. Junge/R. B. Sydnor PHONE: 301-269-4969/4514

LER NO. 83-36/3L
DOCKET NO. 50-318
LICENSE NO. DPR 69
EVENT DATE 07-19-83
REPORT DATE 08-17-83
ATTACHMENT

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (CONT'D)

At 0900 during Surveillance Testing (Functional Trip Tests) of the Reactor Trip Circuit Breakers (TCB) undervoltage (U/V) and shunt trip devices, TCBs 4 and 7 exhibited sluggish trips when tripped by the U/V device. The shunt devices produced satisfactory trips. The response time of TCBs 4 and 7 exceeded that required by T.S. 3.3.1.1 when tripped by the U/V device. TCBs 4 and 7 and their U/V devices were adjusted and tested satisfactorily at 1700.

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

The cause of this event is under investigation. Testing of TCBs 4 and 7 revealed the U/V device's pickup voltage settings and positive trip travel adjustments were outside of specifications. Subsequently, TCBs 1, 2, 3, 5, 6 and 8 were similarly tested and were found to have satisfactory response times, but each of these TCBs had from 1 to 4 adjustments/settings that were outside of specifications similarly to TCBs 4 and 7. Therefore, the cause of the sluggish U/V trip on TCBs 4 and 7 is not clear. All TCBs were adjusted to within required margins, trip shaft and trip latch roller bearings were lubricated, and post-maintenance tested satisfactorily. A new surveillance and preventative maintenance program has been initiated which follows the guidelines of Combustion Engineering's Availability Data Program Infobulletin No. 83-07 of June 15, 1983 and includes the recommendations of General Electric Service Advice letter (SAL) 175 and the supplement to GE SAL 175. The new program will allow for better trending of TCB U/V device problems. All TCBs' U/V devices are being response time tested monthly and out of specification results trended for failure mode determination.

Because the new program has only been in effect for two months, insufficient trend data is available to prove a definitive cause. An update report will be submitted when trend analysis determines the cause of this event.

BALTIMORE GAS AND ELECTRIC COMPANY
P.O. BOX 1475
BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

August 17, 1983

Dr. Thomas E. Murley
Regional Administrator
U. S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

Docket No. 50-318
License No. DPR 69

Dear Dr. Murley:

Attached is LER 83-36/3L, as required per Technical Specification 6.9.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L B Russell
L. B. Russell
Plant Superintendent

LBR:RBS:bsb

cc: Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

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