

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

W. L. STEWART
VICE PRESIDENT
NUCLEAR OPERATIONS

June 7, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Robert A. Clark, Chief
Operating Reactors Branch No. 3
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Serial No. 326
PSE&C/BSD/ses:0498C
Docket Nos. 50-338
50-339
License Nos. NPF-4
NPF-7

Gentlemen:

GENERAL DESIGN CRITERIA 17 ANALYSIS
NORTH ANNA UNIT NOS. 1 AND 2

The purpose of this letter is to respond to your letter of January 11, 1983, Serial No. 040, and attached Safety Evaluation. The two items addressed in the Safety Evaluation are the possible overvoltage on motor control center contactor coils and the resolution of motor operated valve rerating or replacement.

The starter manufacturer was unable to document the acceptability of overvoltage on contactor coils, therefore, Vepco will perform tests of the coils to determine the effects of the overvoltage and determine the resolution required by the results of the tests. The test results and any resolution and related schedule will be submitted when it is available. We presently anticipate this submittal will be available by January 31, 1984. As discussed in our October 26, 1982 letter, Serial No. 608, we believe the overvoltage is not a serious concern.

We have continued our effort to resolve which motor operated valves can be rerated and which may require modification or replacement of the motor operator. Based on information from the motor operator manufacturer 80 of the units can be rerated, one must have the operator replaced and 47 need further information prior to determining if they need to be rerated or replaced.

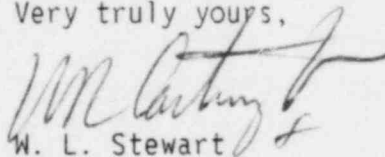
We are continuing our contact with the valve manufacturers in an effort to confirm the torque requirements specified for the MOV's by the motor operator vendor. Response from the valve manufacturers has been much slower than originally expected. We anticipate their response will continue to be slow and that some vendors may never respond as they no longer manufacture nuclear valves. We are initiating engineering of the modification based on the information received from the motor operator vendor. In view of these facts, we estimate the completion of our MOV studies will occur by May 1, 1984. The results of our studies will be provided to you upon completion.

8306160405 830607
PDR ADOCK 05000338
P PDR

A001
1/0

We have previously indicated, in our June 4, 1982 letter, Serial No. 316, to you, our modifications (including torque switch limiter plate installation and MOV replacement when required) are scheduled for completion by the end of the second refueling outage of each unit after September 1, 1982. By October 15, 1983, we will provide you an update on our progress and our plan regarding MOVs for which the MOV manufacturer or valve manufacture has been unable to provide complete information. Until this information is complete, procurement schedules for possible replacement/upgrade cannot be finalized. We will attempt to provide a permanent schedule in a December 15, 1983 update.

Very truly yours,



W. L. Stewart

cc: Mr. R. C. DeYoung, Director
NRC Office of Inspection and Enforcement
Division of Reactor Operations Inspection
Washington, D. C. 20555

Mr. J. P. O'Reilly
Regional Administrator - Region II

Mr. M. E. Shymlock
Resident Inspector - North Anna