

NRC FORM 366 (12-81) 10 CFR 50										U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT										APPROVED BY OMB 3150-0011																																																																					
CONTROL BLOCK:										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																																																																															
[0][1] [M][A][P][P][S][I]										[2][0][0]-[0][0][0][0][0]-[0][0]										[3][4][1][1][1][1]																																																																					
LICENSEE CODE										LICENSE NUMBER										LICENSE TYPE CAT 58																																																																					
CONT										REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE																																																	
[0][1]										[L][6][0][5][0]-[0][2][9][3]										[7][1][0][0][8][8][2]										[8][0][5][1][6][8][3]																																																											
										EVENT DESCRIPTION AND PROBABLE CONSEQUENCES																																																																															
[0][2]										On October 8, 1982 a water leak was observed on the 74' elevation of the reactor																																																																															
[0][3]										building in the RWCU Holding Pump Room. It's origin was a socket weld crack in																																																																															
[0][4]										a 3/4" stainless steel vent line on the RWCU System. The leak was immediately																																																																															
[0][5]										isolated and MR 82-13-34 issued for its repair. The vent line was replaced,																																																																															
[0][6]										tested, and the isolated line returned to service at approximately 0015 hours on																																																																															
[0][7]										October 19, 1982. This event did not threaten the health and safety of the																																																																															
[0][8]										general public.																																																																															
SYSTEM CODE										CAUSE CODE										COMPONENT CODE										COMP SUBCODE										VALVE SUBCODE																																																	
[0][9] [C][G]										[B]										[P][I][P][E][X][X]										[A]										[Z]																																																	
EVENT YEAR										SEQUENTIAL REPORT NO.										OCCURRENCE CODE										REPORT TYPE										REVISION NO.																																																	
[17] [8][2]										[0][4][7]										[0][3]										[X]										[1]																																																	
ACTION TAKEN										FUTURE ACTION										EFFECT ON PLANT										SHUTDOWN METHOD										HOURS										ATTACHMENT SUBMITTED										NPRD-4 FORM SUB.										PRIME COMP. SUPPLIER										COMPONENT MANUFACTURER									
[C]										[Z]										[Z]										[Z]										[0][0][0][0]										[N]										[N]										[A]										[Z][9][9][9]									
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS																																																																																									
[1][0]										Dye penetrant examination revealed other socket weld cracks in the vent assembly.																																																																															
[1][1]										A new assembly was fabricated and installed. Examination of the removed sections																																																																															
[1][2]										revealed that socket connections were bottomed out. The root cause of this																																																																															
[1][3]										event was improper fabrication during original construction.																																																																															
[1][4]																																																																																									
FACILITY STATUS										% POWER										OTHER STATUS										METHOD OF DISCOVERY										DISCOVERY DESCRIPTION																																																	
[1][5] [F]										[0][8][5]										[N/A]										[A]										Operational Event																																																	
ACTIVITY CONTENT RELEASED OF RELEASE										AMOUNT OF ACTIVITY										LOCATION OF RELEASE																																																																					
[1][6] [Z]										[Z]										[N/A]										[N/A]																																																											
PERSONNEL EXPOSURES NUMBER										TYPE										DESCRIPTION																																																																					
[1][7] [0][0][0]										[Z]										[N/A]																																																																					
PERSONNEL INJURIES NUMBER										DESCRIPTION																																																																															
[1][8] [0][0][0]																				[N/A]																																																																					
LOSS OF OR DAMAGE TO FACILITY TYPE										DESCRIPTION																																																																															
[1][9] [Z]																				[N/A]																																																																					
PUBLICITY ISSUED										DESCRIPTION																																																																															
[2][0] [N]																				[N/A]																																																																					
																														NRC USE ONLY																																																											

IF 22 VI

8306010275 830516
PDR ADOCK 05000293
S PDR

J. Vender

PHONE (617) 746-7900

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

May 16, 1983

BECO. Ltr. #83-128

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19403

Docket No. 50-293
License DPR-35

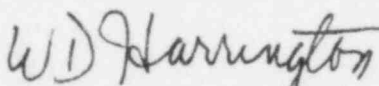
Dear Sir:

The attached Update Licensee Event Report 82-047/03X-1 entitled "RWCU 3/4" Vent Crack", is hereby submitted in accordance with the requirements of Pilgrim Nuclear Power Station Technical Specifications 6.9.8.2.b.

This update is to clarify the cause of the event.

If there are any questions on this subject, please contact us.

Respectfully submitted,


William D. Harrington

WDH/mg

Enclosure: LER 82-047/03X-1 Update

cc: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Standard BECO. LER Distribution

IEU 11