



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report No.: 50-416/83-28

Licensee: Mississippi Power and Light Company
Jackson, MS 39205

Docket Nos.: 50-416

License Nos.: NPF-13

Facility Name: Grand Gulf

Inspection at Grand Gulf site near Port Gibson, Mississippi

Inspector:

A. G. Debbage

7/29/83
Date Signed

Approved by:

C. M. Upright
C. M. Upright, Section Chief
Management Programs Section
Engineering Programs Branch
Division of Engineering and Operational Programs

8/1/83
Date Signed

SUMMARY

Inspection on June 27 - July 1, 1983

Areas Inspected

This routine, unannounced inspection involved 35 inspector-hours on site in the areas of test and measurement equipment program, maintenance program, and non-routine reporting program.

Results

Of the three areas inspected, no violations or deviations were identified.

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REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *W. Angle, Principal Engineer, Operational Analysis
- *J. Bailey, Compliance Coordinator
- J. Bowman, ISC Technician
- *S. Keith, Supervisor, Operations QA
- *C. Hayes, Superintendent, Plant Quality
- J. Holder, Supervisor, I&C
- R. Hutchinson, Manager, Nuclear Support
- *P. Hughes, Corporate Regulatory Compliance
- R. Jain, Supervisor, Reactor Systems Behavior
- *C. McCoy, Plant Manager
- *T. Reeves Jr., Manager of Quality Assurance
- D. Williams, Document Review Coordinator

NRC Resident Inspector

A. Wagner, Senior Resident

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on July 1, 1983, with those persons indicated in paragraph 1 above.

3. Licensee Action on Previous Enforcement Matters

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Test and Measurement Equipment Program (61724)

- References:
- (a) 10 CFR 50 Appendix B, Criterion XII, Control of Measuring and Testing Equipment (M&TE)
 - (b) Regulatory Guide 1.33, Quality Assurance Program Requirements (Operations)
 - (c) ANSI N18.7-1976, Administrative Controls and Quality Assurance Program for the Operational Phase of Nuclear Power Plants
 - (d) MPL-TOP-1A, Policy 12, Control of Measuring and Test Equipment, Revision 2

The inspector reviewed implementing procedures to verify that they met requirements of the accepted QA Program commitments. The calibration laboratory and M&TE issue facility were inspected to verify the following aspects of the M&TE program:

- Criteria and responsibility for assignment of the calibration /adjustment frequency have been established.
- An equipment inventory list has been prepared which identifies equipment used on safety-related structures, systems, or components and the calibration frequency of each piece of equipment.
- Requirements exist for marking the latest calibration date on each piece of equipment.
- A system has been provided for assuring that equipment is calibrated before the date required.
- Requirements have been established which prohibit use of equipment which has not been calibrated within the prescribed frequency.
- Calibration controls have been established which require evaluation of the cause of equipment found out of calibration and the acceptability of items calibrated using such equipment.
- New equipment will be added to the inventory list and calibrated prior to being placed in service.

The implementing procedures reviewed included the following:

07-S-01-5, Calibration and Control of Measuring and Test Equipment, Revision 13
 07-S-43-84, Calibration of Gould DC Pre-Amplifier, Revision 1
 07-S-43-100, Calibration of Dial Indicator, Revision 4
 07-S-43-104, Calibration of the Fluke 8500A Digital Multimeter, Revision 3
 07-S-43-162, Calibration of Honeywell 1858 Visicorder, Revision 0
 07-S-43-163, Calibration of Gould High Voltage Amplifier, Revision 1

The M&TE laboratory is staffed with five technicians and the M&TE issue facility provides 24-hour site support. Approximately 2500 devices are controlled by the M&TE program and the current rate of calibration is 100/125 devices per week. The following equipment items in the laboratory were examined:

MP&L #1662	Ammeter
#1068	Pressure Gauge
#3187	Readout Assembly
#3507	1858 Visicorder
#5043	G 160 Plug Gage

It was observed that all had been returned on the calibration due date.

Within this area, no violations or deviations were identified.

6. Maintenance Program (62702)

- References:
- (a) 07-S-01-1, Control of Maintenance Section Directives, Revision 5
 - (b) 07-S-01-6, Conduct of Maintenance Activities, Revision 1
 - (c) 01-S-07-1, Control of Work on Plant Equipment and Facilities, Revision 9
 - (d) 01-S-07-15, Preventive Maintenance Program, Revision 0.

The inspector reviewed the references and other implementing procedures to verify the following aspects of the maintenance program:

- Written procedures were established for initiating requests for routine and emergency maintenance.
- Criteria and responsibilities for review and approval of maintenance requests were established.
- Criteria and responsibilities that form the basis for designating the activity as safety or non-safety-related were established.
- Criteria and responsibilities were designated for performing work inspection and maintenance activities.
- A written preventive maintenance program for safety-related structures, systems, and components has been established.
- Administrative controls for special processes have been established.
- Method and responsibilities for equipment control have been defined.

Three audit reports on maintenance activities conducted during 1983 were examined.

Audit MAR 83/0051, Control of Preventive Maintenance Scheduling, was conducted May 2-June 9, 1983. The scope of the audit was to compare the preventive maintenance schedule frequency with the vendors' recommendations. Items reviewed were the standby diesel generators, high pressure core spray (HPCS) diesel generators, the reactor core isolation cooling turbine, and standby service water pumps and motors. A problem was identified with the HPCS diesel generator program and a corrective action request CAR 2019 was written; subsequent corrective action was found satisfactory.

Audit MAR 83/0039, Maintenance, was conducted April 14-18, 1983. The scope was to verify that samples of diesel engine cooling water are receiving the required monthly analysis and that improper chemical levels are promptly identified and corrected. No deficiencies were identified.

Audit MAR 83/0025 was conducted March 3-April 11, 1983, to verify that maintenance work activities in progress are being checked by supervisory personnel. The commitments in ANSI N18.7-1976 were reviewed to verify incorporation into plant procedures.

Within this area, no violations or deviations were identified.

7. Nonroutine Reporting Program (90714)

References: (a) 10 CFR 21, Reporting of Defects and Noncompliances
 (b) 10 CFR 50.50, Conditions of Construction Permits
 (c) 10 CFR 50.72, Notification of Significant Events
 (d) Regulatory Guide 1.16, Reporting of Operating Information - Appendix A Technical Specifications, Revision 4.
 (e) Technical Specifications

The inspector reviewed the licensee's reporting program required by References (a) - (e) and verified that reporting requirements were conducted in accordance with regulatory requirements, industry guides or standards, and Technical Specifications. The following criteria were used during this review:

- Administrative controls have been established for prompt review of off-normal events to assure identification of safety-related events.
- Administrative controls have been established for prompt review of planned and unplanned maintenance and surveillance testing activities to assure identification of prospective or actual violations of Technical Specification requirements.
- Administrative controls have been established for completion of corrective actions relating to safety-related operating events.
- Administrative controls have been established for reporting safety-related events internally and to the NRC.
- Administrative controls have been established for recognition and reporting of events covered by 10 CFR 21.
- Administrative controls have been established for review and evaluation of vendor bulletins and circulars.

The following implementing procedures were reviewed to verify that the listed criteria had been incorporated into reporting activities:

01-S-01-11,	Plant Safety Review Committee
01-S-03-1,	GGNS Quality Program, Revision 4
01-S-03-2,	Plant Quality Deficiency Reports, Revision 6
01-S-03-5,	Potential Reportable Deficiency Screening, Revision 1
01-S-06-5,	Incident Reports/Reportable Events

01-S-06-9, GGNS Plant Reporting Requirements to Outside Agencies
 01-S-06-13, IE/INPO/NSAC Documents, Revision 2
 01-S-15-2, Plant Staff Handling of Plant Licensing Activities,
 Revision 0
 01-S-15-3, IE/INPO Documents, Revision 0
 09-S-01-5, Reportable Occurrences
 NPEAP 01-701, Onsite Document Review, Revision 3
 NPEAP 01-702, Offsite Document Review, Revision 2

The method of handling incident reports (IR) was examined. All personnel are held responsible for initiating an IR and handing it to the shift superintendent who transmits to the Operations Superintendent all incidents notified on his shift. He determines if the event is NRC reportable and, if so, contacts the Operations Superintendent, the on-call manager, the NRC regional office, and the NRC resident inspector. The IR is carried to the Shift Technical Advisor (STA) for determination of subsequent NRC reporting. Copies of the IR are sent to the Licensing Engineer, Operations Superintendent, and on-call manager. The on-call manager determines NRC reportability if the STA is not present and makes the necessary reports. After technical evaluation of the IR by the STA, a copy of the IR and evaluation is sent to Plant Quality for their review and for issue of quality deficiency reports or nonconformance reports as appropriate. A copy is also sent to the nuclear plant engineering operations analysis section (OAS) for independent review. Results of the review are sent to the onsite safety review committee (SRC) and recommendations from OAS are sent to training. OAS is also responsible for evaluating and acting on of IE Bulletins, Circulars, Information Notices, INPO documents, and vendor service letters. The offsite Plant Safety Review Committee and SRC have responsibility for reviewing all IRs dealing with NRC reportable occurrences as well as other activities having nuclear safety significance.

Within this area, no violations or deviations were identified.