

April 22, 1991

Mr. A. Bert Davis Regional Administrator U.S. Nuclear Regulatory Commission Region III 799 Roosevelt Road Glen Ellyn, II 60137

Subject:

CECo Braidwood Unit 2 First Outage

Steam Generator Inservice Inspection Results

NRC Docket No. 50-457

Dear Mr. Davis:

Attached is the report of the Steam Generator Eddy Current Examination performed at Braidwood Station. In accordance with Braidwood Technical Specification 4.4.5.5.b and 6.9.2, the complete steam generator tube inservice inspection results shall be submitted to the Commission within 12 months following completion of the inspection. The Steam Generator Eddy Current Surveillance was completed April 26, 1990.

All tubing in each of the four steam generators received full length inspection using the bobbin coil technique. All indications found during bobbin coil inspection, that were not clearly discernible, were also examined with Rotating Pancake Coil Technique to help categorize them as relevant or irrelevant indications.

Attachment A is a summary of the number of tubes plugged in each steam generator. Attachment B begins with a guide to the abbreviations used in the report section. This is then followed by a listing of all indications found, sorted by steam generator.

If there are any questions regarding this information, please contact Mr. Mike Sears at Braidwood Station at (815) 458-2801 extension 2251.

Sincerely,

A.R. Checca

Nuclear Licensing Administrator

Attachment

cc: R. Pulsifer - NRR W. Shaffer - RIII

Resident Inspector - Braidwood NRC Document Control Desk

/scl:ID627:2

9104260136 910422 PDR ADOCK 05000457

50010

IEO!