Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

September 15, 1994
LIC-94-0179

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: August 1994 Monthly Operating Report (MOR)

Enclosed please find the August, 1994 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/dll

Enclosures

c: LeBoeuf, Lamb, Greene & MacRae
L. J. Callan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

IE24
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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

AUGUST 1994
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of August, Fort Calhoun Station (FCS) operated at a nominal 100% power level. Normal plant maintenance, surveillance, and equipment rotation activities occurred during the month, in addition to scheduled online modification activities. The installation of the new higher capacity spent fuel storage racks was completed.

The Fuel Reliability Indicator continues a slowly increasing trend which indicates the likelihood of one (or more) fuel pin leak(s). OPPD is pursuing actions to be taken during the next scheduled refueling outage.

The following NRC inspections were completed during this reporting period:

IER No.	Description
94-17	Monthly Resident Inspection
94-20	Special Inspection regarding the May 26, 1994 Hydrazine Spill

There were no Licensee Event Reports submitted during this reporting period.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of August, no PORV or primary safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

The Reactor Coolant System leak rate was relatively steady throughout the month of August. The leak rate was a nominal .10 gpm with no degrading trends noted. The changes observed for this cycle were due to periodic increases from charging pump packing leaks.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
163	This amendment revised the surveillance test frequencies from monthly to quarterly for several functional tests for reactor protective system and engineered safety feature instrumentation and controls based on Generic Letter 93-05.
164	This amendment made changes to the Technical Specifications to reflect the relocation of the old 10 CFR 20.106 requirements to the new 10 CFR 20.1302. This amendment also implemented administrative changes.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF AUGUST 1994

- Replaced valve on Charging Pump CH-1B Suction Accumulator CH-26B due to accumulator valve core leak.
- Replaced packing on Charging Pump CH-1B.
- Rebuilt Component Cooling Water (CCW) inlet relief valve AC-337 for Charging Pump CH-1B oil cooler due to valve leakage.
- Replaced CCW inlet relief valve AC-283 for Containment Cooler VA-1A cooling coil due to valve leakage.
- Repaired stuck Steam Trap ST-16 on Auxiliary Feedwater Pump FW-10.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE SEPTEMBER 09, 1994
COMPLETED BY D. L. LIPPY
TELEPHONE 402-533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: AUGUST 1994

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:

N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:

N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	5831.0	183505.0
12. Number of Hours Reactor was Critical	744.0	5797.2	143488.9
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	5782.1	141845.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113095.6	8541906.0	187827705.6
17. Gross Elec. Energy Generated (MWH)..	366110.0	2860108.0	61948132.2
18. Net Elec. Energy Generated (MWH)....	348853.6	2728362.5	59101425.4
19. Unit Service Factor.....	100.0	99.2	77.3
20. Unit Availability Factor.....	100.0	99.2	77.3
21. Unit Capacity Factor (using MDC Net)	98.1	97.9	69.8
22. Unit Capacity Factor (using DER Net)	98.1	97.9	68.1
23. Unit Forced Outage Rate.....	.0	.8	4.1

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):

NONE.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	SEPTEMBER 09,1994
COMPLETED BY	D. L. LIPPY
TELEPHONE	402-533-6843

MONTH AUGUST 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	467	17	471
2	464	18	469
3	464	19	467
4	466	20	468
5	468	21	469
6	470	22	470
7	470	23	470
8	470	24	469
9	469	25	468
10	469	26	467
11	471	27	467
12	471	28	467
13	472	29	468
14	473	30	469
15	472	31	470
16	472		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE September 9, 1994
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH August 1994

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
None									During August 1994, the plant operated at a nominal 100% power.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

(9/77)

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending August 31, 1994

1. Scheduled date for next refueling shutdown.	March 11, 1995
2. Scheduled date for restart following refueling.	April 29, 1995
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment?	No
a. If answer is yes, what, in general, will these be?	N/A
b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.	No
c. If no such review has taken place, when is it scheduled?	Prior to April 1995
4. Scheduled date(s) for submitting proposed licensing action and support information.	No submittal planned
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	*
6. The number of fuel assemblies: a) in the core	133 Assemblies
b) in the spent fuel pool	570 Assemblies
c) spent fuel pool storage capacity	1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	2007 Outage

* OPPD is utilizing the CASMO-3/SIMULATE-3 codes for reactor physics related analyses for Cycle 16.

Prepared by Kevin Hoelscher Date 9-9-94