

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of	)	
	)	Dockets Nos. 50-321 and 50-356
GEORGIA POWER COMPANY, ET AL	)	
	)	
(Edwin I. Hatch Nuclear Plant,	)	
Units 1 and 2)	)	
	)	

REVISED ORDER CONFIRMING LICENSEE COMMITMENTS  
ON POST-TMI RELATED ISSUES

I.

The Georgia Power Company (GPC or the licensee) and three other co-owners are the holders of Facility Operating Licenses Nos. DPR-57 and NPF-5 which authorize operation of the Edwin I. Hatch Nuclear Plant, Units 1 and 2 (Hatch or the facilities) at steady state reactor power levels not in excess of 2436 megawatts thermal for each unit. The facilities are boiling water reactors located at the licensee's site in Appling County, Georgia.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial

additional protection in the operation of nuclear facilities based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or after July 1, 1981. On March 17, 1982, a letter (Generic Letter 82-05) was sent to all licensees of operating power reactors for those items that were scheduled to be implemented from July 1, 1981 through March 1, 1982. Subsequently, on May 5, 1982, a letter (Generic Letter 82-10) was also sent to all licensees of operating power reactors for those items that were scheduled for implementation after March 1, 1982. These letters are hereby incorporated by reference. In these letters each licensee was requested to furnish within 30 days pursuant to 10 CFR 50.54(f) the following information for items which the staff had proposed for completion on or after July 1, 1981:

- (1) For applicable items that have been completed, confirmation of completion and the date of completion,
- (2) For items that have not been completed, a specific schedule for implementation, which the licensee committed to meet, and
- (3) Justification for delay, demonstration of need for the proposed schedule, and a description of the interim compensatory measures being taken.

II.

Georgia Power Company responded to Generic Letter 82-05 by letters dated April 20, June 7 and 11, 1982, December 20, 1982, February 11, 1983 and March 11, 1983. The licensee responded to Generic Letter 82-10 by letter dated June 4, 1982. The licensee had previously informed the staff, in a letter dated October 1, 1981 that it had completed the requirement of Item II.D.1.2 concerning submittal of safety relief valve test reports. In the submittals responding to Generic Letters 82-05 and 82-10, the licensee confirmed which of the other items identified in the Generic Letters had been completed and made firm commitments to complete the remainder. The attached Tables summarizing the licensee's schedular commitments or status were developed by the staff from the Generic Letters and the licensee-provided information.

There are six items from Generic Letter 82-10 that, as noted in the Table (Attachment 2), have licensee schedules to be determined and are therefore not included in this Order. Some of the items addressed in this Order are considered by the licensee to be completed or to require no modifications. The staff's evaluation of the remaining items is provided herein:  
Common to Units 1 and 2 is Item II.E.4.2(7) Containment Isolation Dependability, Position 7.

This item is still under review by NRC. As a consequence, this item, scheduled for completion by July 1, 1981, will be the subject of a future NRC action and is therefore not included in this Order.

For Hatch 1, regarding Generic Letter 82-05, there are seven incomplete items: II.B.3, Post Accident Sampling, II.F.1(1) (2), (3) and (4), Post Accident Monitors, and II.K.3.22, RCSD Suction Modification and II.E.4.2, Part 7. The licensee's implementation schedule for Item II.B.3 is given as July 1, 1983. The implementation schedule for Items II.F.1(3) and (4) is given as the end of the outage following Cycle 5b (also referred to as Cycle 6). This outage is scheduled for completion in mid-February 1983. Items II.F.1(1) and (2) can be completed with the reactor in operation and are scheduled for completion by March 15, 1983. Item II.K.3.22, for which equipment delivery is expected in June 1983, can also be completed with the reactor in operation and the proposed schedule is September 30, 1983. The reasons given for delaying II.B.3 are that the NRC issuance of Regulatory Guide 1.97 imposed additional requirements which resulted in a significant delay in the system design, a supplier experienced a work stoppage and was unable to meet a scheduled delivery date, and recently identified concerns related to the chloride analysis portion of the sampling system. The remaining five items for Hatch 1 are being impacted by procurement and documentation delays that have resulted in the completion schedule being delayed as described above. In our judgment, these delays were beyond the control of GPC. For Hatch 2, there are four incomplete items: II.B.3, II.F.1(1) and (2) and II.K.3.22; these items are included in the seven, discussed above for Hatch 1. The licensee's implementation schedule for Hatch 2 is given as July 1, 1983 for Item II.B.3, the end of the outage following Cycle 3 for Items II.F.1(1) and (2), and September 30, 1983, for Item II.K.3.22. The Hatch 2 Cycle 4 reload is scheduled for Spring 1983. The reasons given for delaying the Hatch 2 items

are the same as for the congruent Hatch 1 items. Therefore, we conclude the delays are justified.

The licensee has provided for interim compensatory measures for all of the items that require a plant hardware modification. In regard to each such hardware item, the compensatory measures are as follows:

II.B.3 - Two existing systems in place which can provide for adequate post-accident sampling of both reactor coolant and drywell atmosphere.

II.F.1(1) - Interim systems to quantify high level releases and in-plant iodine are in place.

II.F.1(2) - Same as for II.F.1(1).

II.F.1(3) - Post accident gamma radiation monitors installed in the original plant design are available as interim monitoring devices.

II.F.1(4) - Drywell pressure is currently indicated on a recorder which serves as an interim monitoring device.

II.K.3.22 - Manual transfer procedures in place, as interim measure, until automatic switchover hardware in operation.

We find, based on the above evaluation, that: 1) the licensee has taken corrective actions regarding the delays and has made a responsible effort to implement the NUREG-0737 requirements noted; 2) there is good cause for the several delays (unexpected design complexity, interface problems, and equipment delays); and 3) as noted above, interim compensatory measures have been provided.

In view of the foregoing, I have determined that these modifications and actions are required in the interest of public health and safety and, therefore, the licensee's commitment should be confirmed by Order.

IV.

Accordingly, pursuant to Sections 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT THE LICENSEE SHALL:

Implement and maintain the specific items described in the Attachments to this Order in the manner described in the licensee's submittals noted in Section III herein no later than the dates in the Attachments.

V.

The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. A request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. A REQUEST FOR A HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

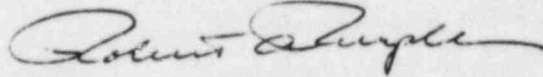
If a hearing is requested by the licensee, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the

requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Purple, Deputy Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland,  
this 30th day of March 1983.

Attachments:

1. Licensee's Commitments on Applicable  
NUREG-0737 Requirements from Generic  
Letter 82-05
2. Licensee's Commitments on Applicable  
NUREG-0737 Requirements from Generic  
Letter 82-10

LICENSEE'S COMMITMENTS ON APPLICABLE NUREG-0737 ITEMS FROM GENERIC LETTER 82-05

Attachment 1

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)	
				Unit 1	Unit 2
P.A.3.1	Simulator Exams	10/1/81	Include simulator exams in licensing examinations	Complete	Complete
II.B.2	Plant Shielding	1/1/82	Modify facility to provide access to vital areas under accident conditions	Complete	Complete
II.B.3	Post-Accident Sampling	1/1/82	Install upgrade post-accident sampling capability	July 1, 1983	July 1, 1983
II.B.4	Training for Mitigating Core Damage	10/1/81	Complete training program	Complete	Complete
II.E.4.2	Containment Isolation Dependability	7/1/81	Part 5-lower containment pressure setpoint to level compatible w/normal operation	Complete	Complete
		7/1/81	** Part 7-isolate purge & vent valves on radiation signal	Technical Exception	Technical Exception

\*\* Not Part of Confirmatory Order

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status)*	
				Unit 1	Unit 2
II.F.1	Accident Monitoring	1/1/82	(1) Install noble gas effluent monitors	March 15, 1983	**
		1/1/82	(2) Provide capability for effluent monitoring of iodine	March 15, 1983	**
		1/1/82	(3) Install in-containment radiation-level monitors	*	Complete
		1/1/82	(4) Provide continuous indication of containment pressure	*	Complete
		1/1/82	(5) Provide continuous indication of containment water level	Complete	Complete
		1/1/82	(6) Provide continuous indication of hydrogen concentration in containment	Complete	Complete
II.K.3.15	Isolation of HPCI & RCIC Modification	7/1/81	Modify pipe break detection logic to prevent inadvertent isolation	Complete	Complete
II.K.3.22	RCIC Suction	1/1/82	Modify design of RCIC suction to provide automatic transfer to torus	Sept. 30, 1983	Sept. 30, 1983
II.K.3.24	Space Cooling for HPCI/RCIC	1/1/82	Confirm adequacy of space cooling for HPCI/RCIC	Complete	Complete
II.K.3.27	Common Reference Level	7/1/81	Provide common Reference Level for Vessel Level Instrumentation	Complete	Complete

\*End of outage followed by Cycle 5b, currently scheduled for completion in mid-February 1983.

\*\*End of outage following Cycle 3, currently scheduled for Spring 1983.

Item	Title	NUREG-0737 Schedule	Requirement	Licensee's Completion Schedule (or status) for Units 1 & 2
I.A.1.3.1	Limit Overtime	10/1/82 per Gen. Ltr. 82-12 dtd. 6/15/82	Revise administrative procedures to limit overtime in accordance w/NRC Policy Statement issued by Generic Ltr. No. 82-12, dtd. June 15, 1982	Complete
I.A.1.3.2	** Minimum Shift Crew	To be superseded by Proposed Rule	To be addressed in the Final Rule on Licensed Operator Staffing at Nuclear Power Units.	To be addressed when Final Rule is issued.
I.C.1	** Revise Emergency Procedures	Superseded by SECY 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be determined
II.D.1.2	RV and SV Test Programs	7/1/82	Submit plant specific reports on relief & safety valve program	Complete
II.K.3.18	ADS Actuation	9/30/82	Submit revised position on need for modifications	Complete
II.K.3.30 & 31	** SBLOCA Analysis	1 year after staff approval of model	Submit plant specific analyses	To be determined following staff approval of model
III.A.1.2	** Staffing Levels for Emergency Situations	Superseded by SECY 82-111	Reference SECY 82-111, Requirements for Emergency Response Capability	To be Determined
III.A.1.2	** Upgrade Emergency Support Facilities	" " "	" " " " " " " "	" " " " " " " "
III.A.2.2	** Meteorological Data	" " "	" " " " " " " "	" " " " " " " "
III.D.3.4	Control Room Habitability	To be determined by licensee	Modify facility as identified by licensee study.	Complete

\*\*Not Part of Confirmatory Order