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SUBJECT: ISPT AND 2 - SAFETY EVALUATION
ISPT PROGRAM (ISPT)

In a letter dated October 1989, the Tennessee Valley Authority (TVA) submitted a request for an Inservice Pressure Test (ISPT) Program with a specific in-service system pressure test program. The associated request for relief was dated August 18, 1983, which was approved on September 3, 1987, and January 1989.

In a letter dated October 1989, TVA submitted its relief request, and for the most part, with one exception. The existing revisions of the American Society of Mechanical Engineers Pressure Vessel Code (code) and related articles (related requirements) provided additional information.

The staff has reviewed the additional information and accepts the use of the specific program. Our safety evaluation is enclosed.

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Enclosure:
As stated

cc w/enclosure: 9011290255 901120
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MEMORANDUM FOR: Jack N. Donohew
Project Directorate 11-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

FROM: Ching-Yao Cheng
Materials and Chemical Engineering Branch
Division of Engineering Technology
Office of Nuclear Reactor Regulation

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2 - SAFETY EVALUATION
REPORT OF INSERVICE PRESSURE TEST PROGRAM (ISPT)

In a letter dated January 31, 1989, the Tennessee Valley Authority (TVA) submitted a revised ASME Code Section XI (Code) Inservice Pressure Test (ISPT) Program with one request for relief (ISPT-1) from certain inservice system pressure test requirements for review. The program and its associated request for relief were originally submitted to the NRC by TVA letter dated August 18, 1983, which was followed by TVA letters dated March 10, 1986, September 3, 1987, and January 11, 1988.

In a letter dated October 11, 1989, the staff reviewed the ISPT Program and its relief request, and found the relief request and the program acceptable with one exception. The exception concerned the use of portions of later revisions of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (code) specific paragraphs without using the entire articles (related requirements). In a letter dated April 5, 1990, TVA provided additional information addressing related requirements.

The staff has reviewed the additional information provided by the licensee and accepts the use of the specific portions of the code utilized by the licensee. Our safety evaluation is enclosed.

Sincerely,
Original Signed by:
C.Y. Cheng

Ching-Yao Cheng
Materials and Chemical Engineering Branch
Division of Engineering Technology
Office of Nuclear Reactor Regulation

Enclosure:
As stated

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