



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated March 28, 1994, as supplemented on June 27, 1994, and July 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-44 paragraph 2.C.(5) is hereby amended to read as follows:
 - 2.C.(5) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated

August 14, September 15, October 10 and November 24, 1980, and in the NRC SER dated September 16, 1993, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. Further, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 194, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Mohan C. Thadani, Acting Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

1. Page 4a of License*
2. Changes to the Technical Specifications

Date of Issuance: August 24, 1994

*Page 4a is attached, for convenience, for the composite license to reflect this change.

- 2.C.(5) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993 and August 24, 1994, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This amended license is subject to the following conditions for the protection of the environment:
- (a) Upon completion of the licensees' "smoke tests," the licensees' shall release all procedures, data, and analysis bearing on the calculation of χ/Q to the Regulatory Staff as soon as possible but no later than initial fuel loading for Peach Bottom Unit 3. Using the "smoke test" results to re-calculate χ/Q and using the other assumptions previously used by the Regulatory Staff (or realistic refinements thereof), the Regulatory Staff shall re-calculate the potential radioiodine thyroid dose to a child via the milk pathway at the points of maximum concentration at or beyond the site boundary where dairy cows are present or could be pastured. If this calculated dose based on the combined operation of Peach Bottom Units 2 and 3 exceeds 15 mrem/year, then the licensees' shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.
- (b) If the actual milk sample measurements taken at the nearby farms in accordance with the technical specifications, predict a dose to a child's thyroid, based on actual combined operation of

ATTACHMENT TO LICENSE AMENDMENT NO. 194

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

| <u>Remove</u> | <u>Insert</u> |
|-------------------|-------------------|
| 240c | 240c |
| 240d through 240s | 240d through 240s |
| 244 | 244 |
| 246 | 246 |
| 248 | 248 |
| 252 | 252 |
| 253 | 253 |

PBAPS

The Fire Protection Limiting Conditions for Operation and Surveillance Requirements have been relocated to the Fire Protection Program, which is part of the UFSAR. These requirements are still mandated by the Facility Operating License.

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PBAPS

Continued

6.2.2 Facility Staff

The facility organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.1, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. Deleted.
- g. In addition to the requirements of 6.2.2.a, the Senior Manager-Operations shall hold a Senior Reactor Operator license.

6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) Senior - Health Physicist (radiation protection manager) who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall meet the qualifications specified by the 1985 NRC Policy Statement on Engineering Expertise on Shift.

6.4 Training

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training and shall meet the requirements of Section 5.5 of ANSI N18.1-1971 and 10 CFR 55, Appendix A.

6.4.2 DELETED

6.5 Review and Audit

6.5.1 Plant Operations Review Committee (PORC)

6.5.1.1 Function

The Plant Operations Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

6.5.1.2 Composition

The Plant Operations Review Committee is composed of nine regular members from Peach Bottom Atomic Power Station staff. Members shall collectively have experience in the following areas:

Plant Operations
 Engineering
 Maintenance
 Instrumentation and Controls
 Planning
 Radiation Safety
 Chemistry
 Experience Assessment

Members shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.4, or 4.6 for applicable required experience and be appointed in writing by the Plant Manager. The Chairman and alternate Chairmen of the PORC shall be drawn from the PORC members and appointed in writing by the Plant Manager.

6.5.1.3 Alternates

Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternate members shall be used to satisfy a PORC quorum (See 6.5.1.5).

PBAPS

6.5.1.6 Continued

- i. Review of the Plant Security Plan, and shall submit recommended changes to the Plan to the Plant Manger and the Nuclear Review Board.
- j. Review of the Emergency Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- k. Review of every unplanned release reportable under 10CFR 50.72 and 50.73, of radioactive material to the environs; evaluate the event; specify remedial action to prevent recurrence; and document the event description, evaluation, and corrective action and the disposition of the corrective action in the plant records.
- l. Review of the facility Fire Protection Program, and shall submit recommended changes to the Program to the Plant Manager and the Nuclear Review Board.

Authority

6.5.1.7 The Plant Operations Review Committee shall:

- a. Recommend in writing, to the Plant Manager, for his approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(b) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide immediate written notification to the Vice President, Peach Bottom Atomic Power Station, or in his absence, the Senior Vice President - Nuclear and the Nuclear Review Board of disagreement between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

PBAPS

6.5.2.8 Continued

- e. The Facility Emergency Plan and implementing procedures at least once per year.
- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. The Offsite Dose Calculation Manual and implementing procedures at least once per two years.
- h. The performance of activities required by the Quality Assurance Program regarding the radiological monitoring program to meet the provisions of Regulatory Guide 4.1, Revision 1, April 1975, at least once per calendar year.
- i. Any other area of facility operation considered appropriate by the NRB or the Executive Vice President - Nuclear.
- j. The facility Fire Protection Program and implementing procedures at least once per year.

Authority

- 6.5.2.9 The NRB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

- 6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Executive Vice President - Nuclear within 10 working days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7.e, f, g, and h above shall be prepared, approved and forwarded to the Executive Vice President - Nuclear, within 10 working days following completion of the review.

PBAPS

6.6 Reportable Event Action

6.6.1 The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified pursuant to the requirements of Section 50.73 to 10 CFR 50.
- b. Each Reportable Event Report submitted to the Commission shall be reviewed by the PORC and submitted to the NRB and the Vice President, Peach Bottom Atomic Power Station.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President, Peach Bottom Atomic Power Station, Plant Manger, and the NRB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the Vice President, Peach Bottom Atomic Power Station within 10 working days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained as follows, except as provided in Technical Specifications 6.8.2 and 6.8.3:

- a. Procedures that meet the requirements of Sections 5.1 and 5.3 of ANSI N18.7-1972
- b. Procedures that meet the requirements of Appendix "A" of USAEC Regulatory Guide 1.33 (November 1972)
- c. Procedures covering radiological effluent technical specification activities, including the Offsite Dose Calculation Manual and Quality Assurance Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.
- d. Procedures for the implementation of the Fire Protection Program.



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DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 198
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated March 28, 1994, as supplemented on June 27, 1994, and July 8, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-56 paragraph 2.C.(4) is hereby amended to read as follows:
 - 2.C.(4) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in

the NRC SER dated September 16, 1993, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. Further, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 198, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Mohan C. Thadani, Acting Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

1. Page 4a of the License*
2. Changes to the Technical Specifications

Date of Issuance: August 24, 1994

*Page 4a is attached, for convenience, for the composite license to reflect this change.

- 2.C.(4) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC SER dated May 23, 1979 and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993 and August 24, 1994 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- 2.C.(5) Operation beyond the end-of-cycle (all rods out condition) thermal power is limited to seventy (70) percent minimum. Increasing core power level via reduced feedwater heating, once operation in the coastdown mode has begun, is not permitted unless the licensee has performed an analysis of this operating condition that confirms that this condition is bounded by the analysis for the particular cycle of operation.

3. This license is subject to the following conditions for the protection of the environment:

- A. If the actual milk sample measurements taken at the nearby farms in accordance with the Technical Specifications, predict a dose to a child's thyroid, based on actual combined operation of Unit 2 and Unit 3, that exceeds 15 mrem/year, licensees shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.

ATTACHMENT TO LICENSE AMENDMENT NO. 198

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

| <u>Remove</u> | <u>Insert</u> |
|-------------------|-------------------|
| 240c | 240c |
| 240d through 240s | 240d through 240s |
| 244 | 244 |
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The Fire Protection Limiting Conditions for Operation and Surveillance Requirements have been relocated to the Fire Protection Program, which is part of the UFSAR. These requirements are still mandated by the Facility Operating License.

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6.2.2 Facility Staff

The facility organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.1, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. Deleted.
- g. In addition to the requirements of 6.2.2.a, the Senior Manager-Operations shall hold a Senior Reactor Operator license.

PBAPS

6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) Senior - Health Physicist (radiation protection manager) who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall meet the qualifications specified by the 1985 NRC Policy Statement on Engineering Expertise on Shift.

6.4 Training

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training and shall meet the requirements of Section 5.5 of ANSI N18.1-1971 and 10 CFR 55, Appendix A.

6.4.2 DELETED

6.5 Review and Audit6.5.1 Plant Operations Review Committee (PORC)6.5.1.1 Function

The Plant Operations Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

6.5.1.2 Composition

The Plant Operations Review Committee is composed of nine regular members from Peach Bottom Atomic Power Station staff. Members shall collectively have experience in the following areas:

Plant Operations
Engineering
Maintenance
Instrumentation and Controls
Planning
Radiation Safety
Chemistry
Experience Assessment

Members shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.4, or 4.6 for applicable required experience and be appointed in writing by the Plant Manager. The Chairman and alternate Chairmen of the PORC shall be drawn from the PORC members and appointed in writing by the Plant Manager.

6.5.1.3 Alternates

Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternate members shall be used to satisfy a PORC quorum (See 6.5.1.5).

PBAPS

6.5.1.6 Continued

- i. Review of the Plant Security Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- j. Review of the Emergency Plan, and shall submit recommended changes to the Plan to the Plant Manager and the Nuclear Review Board.
- k. Review of every unplanned release reportable under 10CFR 50.72 and 50.73, of radioactive material to the environs; evaluate the event; specify remedial action to prevent recurrence; and document the event description, evaluation, and corrective action and the disposition of the corrective action in the plant records.
- l. Review of the facility Fire Protection Program, and shall submit recommended changes to the Program to the Plant Manager and the Nuclear Review Board.

Authority

6.5.1.7 The Plant Operations Review Committee shall:

- a. Recommend in writing, to the Plant Manager, for his approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(b) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide immediate written notification to the Vice President, Peach Bottom Atomic Power Station, or in his absence, the Senior Vice President - Nuclear and the Nuclear Review Board of disagreement between the PORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

PBAPS

6.5.2.8 Continued

- e. The Facility Emergency Plan and implementing procedures at least once per year.
- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. The Offsite Dose Calculation Manual and implementing procedures at least once per two years.
- h. The performance of activities required by the Quality Assurance Program regarding the radiological monitoring program to meet the provisions of Regulatory Guide 4.1, Revision 1, April 1975, at least once per calendar year.
- i. Any other area of facility operation considered appropriate by the NRB or the Executive Vice President - Nuclear.
- j. The facility Fire Protection Program and implementing procedures at least once per year.

Authority

- 6.5.2.9 The NRB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

- 6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Executive Vice President - Nuclear within 10 working days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7.e, f, g, and h above shall be prepared, approved and forwarded to the Executive Vice President - Nuclear, within 10 working days following completion of the review.

PBAPS

6.6 Reportable Event Action

6.6.1 The following actions shall be taken for Reportable Events:

- a. The Commission shall be notified pursuant to the requirements of Section 50.73 to 10 CFR 50.
- b. Each Reportable Event Report submitted to the Commission shall be reviewed by the PORC and submitted to the NRB and the Vice President, Peach Bottom Atomic Power Station.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President, Peach Bottom Atomic Power Station, Plant Manger, and the NRB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the Vice President, Peach Bottom Atomic Power Station within 10 working days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained as follows, except as provided in Technical Specifications 6.8.2 and 6.8.3:

- a. Procedures that meet the requirements of Sections 5.1 and 5.3 of ANSI N18.7-1972
- b. Procedures that meet the requirements of Appendix "A" of USAEC Regulatory Guide 1.33 (November 1972)
- c. Procedures covering radiological effluent technical specification activities, including the Offsite Dose Calculation Manual and Quality Assurance Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.
- d. Procedures for the implementation of the Fire Protection Program.