

CONTROL BLOCK										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																											
01	N	C	M	G	S	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5											
LICENSEE CODE					LICENSEE NUMBER										LICENSEE TYPE					CAT 15																	
CONT																																					
01	L	5	0	5	0	0	0	3	6	9	7	0	6	0	2	8	3	8	0	7	0	6	8	3	9												
REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE							
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																																					
02	While in Mode 3, during a reactor trip transient (reactor had tripped due to																																				
03	steam generator 1B Lo-Lo level), main steam code safety relief valve 1SV-9 lifted																																				
04	while the main steam pressure (app. 1121 psig) was below the acceptable lift																																				
05	setpoint range (1179 to 1202 psig). The valve was declared inoperable per T.S.																																				
08	3.7.1.1 which is reportable per T.S.6.9.1.1(b) and similar to RO-370/83-12.																																				
07	There were still 4 operable code safety valves on S/G 1C, and 1SV-9 was still																																				
09	capable of relieving steam pressure. Health and safety of the public were																																				
	unaffected.																																				
09	C	C	11	E	12	B	13	V	A	L	V	E	X	14	J	15	B	16																			
SYSTEM CODE				CAUSE CODE				CAUSE SUBCODE				COMPONENT CODE				COMP SUBCODE				VALVE SUBCODE																	
17	8	3						0	3	7				0	3																						
EVENT YEAR				SEQUENTIAL REPORT NO.				OCCURRENCE CODE				REPORT TYPE				REVISION NO.																					
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NRC-4 FORM SUB		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER																					
E	18	Z	19	Z	20	Z	21	0	0	0	0	N	23	N	24	L	25	C	7	1	0																
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																																					
10	Apparently the valve's (Crosby, nozzle type C safety valve, style HA-65W) set-																																				
11	point drifted from its original setting of 1200 psig. The adjusting nut is held																																				
12	stationary with a cotter pin, however minor drift (possibly caused by vibration																																				
13	and heat) is not uncommon. All power range high flux setpoints were adjusted																																				
14	per T.S.3.7.1.1 action a, and steam pressure was kept well below the lift set																																				
	point until the setpoint was adjusted to its correct value.																																				
15	X	28	0	0	0	29	Mode 3	A	31	Control Room Alarms/Indications																											
FACILITY STATUS				% POWER				OTHER STATUS				METHOD OF DISCOVERY				DISCOVERY DESCRIPTION																					
ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY																																	
16	Z	33	Z	34	N/A																																
PERSONNEL EXPOSURES				PERSONNEL INJURIES				LOSS OF OR DAMAGE TO FACILITY				PUBLICITY ISSUED				NRC USE ONLY																					
NUMBER				TYPE				DESCRIPTION				DESCRIPTION				DESCRIPTION																					
17	0	0	0	37	Z	38	N/A																														
18	0	0	0	40			N/A																														
19	Z	42					N/A																														
20	N	44					N/A																														
NAME OF PREPARER																																					
Phillip B. Nardoci																																					
PHONE																																					
(704) 373-7432																																					

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HAL B. TUCKER
VICE PRESIDENT
NUCLEAR PRODUCTION

83 JUL 12 A9:23

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July 6, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street NW, Suite 2900
Atlanta, Georgia 30303

Re: McGuire Nuclear Station Unit 1
Docket No. 50-369

Dear Mr. O'Reilly:

Please find attached Reportable Occurrence Report RO-369/83-37. This report concerns T.S. 3.7.1.1, "All main steam line code safety valves associated with each steam generator shall be operable with lift settings as specified in Table 3.7-3". This incident was considered to be of no significance with respect to the health and safety of the public.

Very truly yours,

H.B. Tucker 1/5/83
Hal B. Tucker

PBN:jfw
Attachment

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Records Center
Institute of Nuclear Power Operations
1100 Circle 75 Parkway, Suite 1500
Atlanta, Georgia 30339

Mr. W. T. Orders
NRC Resident Inspector
McGuire Nuclear Station

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