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August 22, 1994

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Request for Approval of Revision 6 of NSPNAD 8102-A, Prairie Island Nuclear
Power Plant Reload Safety Evaluation Methods for Application to PI Units

The currently approved reload safety evaluation methods for Prairie Island are described in Revision 5 of NSPNAD-8102-A, "Prairie Island Nuclear Power Plant Reload Safety Evaluation Methods For Application to PI Units". The purpose of this letter is to submit Revision 6 of NSPNAD-8102-A for NRC approval.

The minimum Departure from Nucleate Boiling Ratio (DNBR) limits used for accidents and transients are dependent on the Critical Heat Flux (CHF) correlations that are used. The two CHF correlations incorporated in Revision 5 are designated as WRB-1 and W-3. The changes proposed in Revision 6 are limited to modification of CHF correlations used to analyze the steam line break accidents only. These changes are as follows:

1. Remove reference to the WRB-1 correlation
2. Define the range of the W-3 correlation and the associated DNBR Limit as shown below:

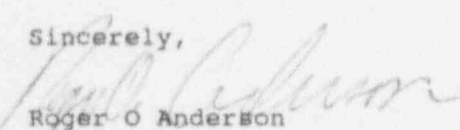
<u>Pressure Range (psia)</u>	<u>DNBR Limit</u>
>1000	1.30
500-1000	1.45

(The basis for this definition is provided by Reference 1.)

Attachment 1 provides a markup of the four affected pages of the topical (pages 184, 188, 189, and 261) and Attachment 2 provides typed versions of these pages, which constitutes Revision 6. It is noted that the new Reference 7, which has been added on page 261, is the letter that transmitted the NRC documents described in Reference 1 to this letter.

If you have any questions or comments please contact Mel Opstad at 612-295-1653.

Sincerely,


Roger O Anderson
Director
Licensing and Management Issues

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USNRC

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c: Regional Administrator III, NRC
Senior Resident Inspector, NRC
NRR Project Manager, NRC
J E Silberg

Reference:

1. NRC's Safety Evaluation of the Westinghouse Electric Corporation Topical Report WCAP-9226-P/9227-NP, "Reactor Core Response To Excessive Secondary Steam Releases" that was transmitted via a Jan 31, 1989 letter from Ashok C. Thadani to Mr. W. J. Johnson