

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | 1 | L | L | S | C | 1 | 2 | 0 | 1 | 0 | - | 0 | 1 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 0 | 0 | 1 | 0 | 4 | 5
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 56

CONT

01 | REPORT SOURCE | L | 5 | 0 | 5 | 0 | 0 | 0 | 3 | 7 | 1 | 3 | 7 | 0 | 5 | 1 | 2 | 1 | 0 | 8 | 1 | 3 | 3 | 0 | 6 | 0 | 8 | 8 | 1 | 3 | 9
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36
 DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | During evaluation of the turbine bypass valve system response time requirements
 03 | specified in Tech. Spec. 4.7.10.b.3, it was determined that data from LIS-EH-03 had
 04 | been improperly evaluated. Turbine bypass valves #2 and #5 did not fast open. Three
 05 | of the five valves opened within required limits. Safe plant operation was maintained
 06 | at all times.

07 |
 08 |
 09 |

09 | SYSTEM CODE | H | E | 11 | CAUSE CODE | X | 12 | CAUSE SUBCODE | C | X | 13 | COMPONENT CODE | Z | Z | Z | Z | Z | 14 | COMP. SUBCODE | Z | 15 | VALVE SUBCODE | Z | 16 |
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47
 17 | LER/RO REPORT NUMBER | 8 | 3 | 21 | 22 | SEQUENTIAL REPORT NO. | 0 | 4 | 9 | 24 | 25 | OCCURRENCE CODE | 0 | 3 | 28 | 29 | REPORT TYPE | L | 30 | REVISION NO. | 0 | 32 |
 ACTION TAKEN | B | 18 | FUTURE ACTION | G | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 0 | 37 | 40 | ATTACHMENT SUBMITTED | Y | 41 | NPD-4 FORM SUBL. | N | 42 | PRIME COMP. SUPPLIER | Z | 43 | COMPONENT MANUFACTURER | Z | 9 | 9 | 9 | 44 | 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | Control circuitry for valves #2 and #5 had been wired incorrectly. The circuits were
 11 | rewired, and the valves were found to be within required limits. Incorrect data
 12 | evaluation stemmed from misinterpretation of the flow to valve position ratio. When
 13 | the flow/valve position is determined, procedure changes will be made stating required
 14 | valve position.

15 | FACILITY STATUS | B | 28 | % POWER | 0 | 0 | 0 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | C | 31 | DISCOVERY DESCRIPTION | SYSTEM EVALUATION | 32 |
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
 16 | ACTIVITY CONTENT | Z | 33 | RELEASED OF RELEASE | Z | 34 | AMOUNT OF ACTIVITY | NA | 35 | LOCATION OF RELEASE | NA | 36 |
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
 17 | PERSONNEL EXPOSURES | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | NA | 39 |
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
 18 | PERSONNEL INJURIES | 0 | 0 | 0 | 40 | DESCRIPTION | NA | 41 |
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
 19 | LOSS OF OR DAMAGE TO FACILITY | Z | 42 | TYPE | NA | 43 |
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
 20 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | NA | 45 |
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

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 PDR ADOCK 05000373
 S PDR

NRC USE ONLY

NAME OF PREPARER

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June 8, 1983

Mr. James G. Keppler
Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Sir:

Reportable Occurrence Report #83-049/03L-0 Docket #050-373 is being submitted to your office in accordance with LaSalle County Nuclear Power Station Technical Specification 6.6.B.2.(a) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/GW/rg

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC Document Management Branch
Inpo-Records Center
File/NRC

JUN 20 1983

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- I. LER NUMBER: 83-049/03L-0
- II. LASALLE COUNTY STATION: Unit 1
- III. DOCKET NUMBER: 050-373
- IV. EVENT DESCRIPTION:

On May 20, 1983, at 0830 hours during an evaluation of whether the turbine bypass valve system could meet the response time requirements as specified in Technical Specifications 4.7.10.b.3, it was noted that the data taken for LIS-EH-03, Turbine Bypass System Response Time Test, on June 17, 1982, had been improperly evaluated and that the #2 and #5 turbine bypass valves did not meet the 200 msec response time limit.

V. PROBABLE CONSEQUENCES OF THIS OCCURRENCE:

Three of the five turbine bypass valves opened within the required limits of the Technical Specifications. The response times of the other two were less than 300 msec. The bypass system was always capable of proper response to high reactor pressure. Safe plant operation was maintained at all times.

VI. CAUSE:

After evaluation of turbine bypass valve control circuitry it was found that the #2 and #5 turbine bypass valves had been wired incorrectly. An "a" contact in the #2 BPV logic was incorrectly wired as a "b" contact. A "b" contact in the #5 BPV logic was incorrectly wired as an "a" contact.

The improper evaluation of the data taken for LIS-EH-03 on June 17, 1982, stemmed from personnel error coupled with misinterpretation of the rated flow to valve position ratio as a linear relationship, i.e., correlation of 80% rated flow with 80% valve position.

VII. CORRECTIVE ACTION:

#2 and #5 turbine bypass valves were rewired under Work Request L24818. The valves were then tested under LST-83-61, BPV Response Time Test, and were found to be within the 200 msec limit. All the turbine bypass valves are full open in less than 200 msec.

To prevent future misinterpretation of 80% flow position as 80% valve position, an AIR #1-1-83-67014 has been initiated to change LIS-EH-03 in accordance with the results of STP-26, Relief Valves. An AIR, #1-1-83-67013, was also in effect to change Technical Specifications 4.7.10.b.3 in accordance with STP-26 results within 90 days of test completion.

Prepared by: Kenneth J. Kalmon