Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

August 15, 1994
LIC-94-0166

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: July 1994 Monthly Operating Report (MOR)

Enclosed please find the July, 1994 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/dll

Enclosures

c: LeBoeuf, Lamb, Greene & MacRae
L. J. Callan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

JULY 1994
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of July, Fort Calhoun Station (FCS) operated at a nominal 100% power level. Normal plant maintenance, surveillance, and equipment rotation activities occurred during the month, in addition to scheduled online modification activities.

Work continued on the installation of new Spent Fuel Pool racks, including numerous fuel movements within the Spent Fuel Pool. The last of the old racks was removed from the pool during the month of July.

During the July 4 nightshift, FCS was notified of the inadvertent actuation of one emergency notification siren caused by thunderstorm activity. The appropriate notifications were made to local and state officials, and to the NRC resident inspector. On July 5, thunderstorms caused inadvertent actuation of two additional emergency sirens. Again, appropriate notifications were made and repairs completed.

Modification work is underway outside the protected area on the 161 kV electrical system. An additional line is being routed from Omaha in support of a major construction project (Cargill) north of the Fort Calhoun Station. Installation of the poles for the new 161 kV line continued throughout July. On July 29, 1994, a contract worker was injured when a pole for the 161 kV line rolled off the back of a truck onto his leg. The accident took place on OPPD owner-controlled property, outside the switchyard. A helicopter ambulance was summoned and local media personnel arrived. Although the incident was outside the switchyard and the protected area, OPPD provided a 4-hour notification to the NRC Operations Center due to the potential for a press release and media interest in a personnel injury accident at the Fort Calhoun Station.

The Fuel Reliability Indicator has been exhibiting a slowly increasing trend that may be indicative of a fuel pin leak.

The following NRC inspection was completed during this reporting period:

IER No.	Description
94-09	Maintenance Reliability Initiative Team Inspection

There were no Licensee Event Reports (LERs) submitted during this reporting period.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of July, no PORV or primary safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

The Reactor Coolant System (RCS) leak rate was relatively steady throughout the month of July. The leak rate was a nominal 0.10 gpm with no degrading trends noted. The only changes observed for this cycle were due to normal plant transients and periodic increases from charging pump packing leaks.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No. Description

NONE

5. SIGNIFICANT SAFETY RELATED MAINTENANCE

- Replaced Inverter "B" Bypass Transformer EE-4P.
- Replaced outboard bearing on Component Cooling Water Pump AC-3A due to increased noise from the outboard bearing.
- Replaced leaking packing and installed new plungers and plunger rings on Charging Pump CH-1C.
- Charging Pump CH-1A Discharge Drain Valve CH-259 was leaking by seat. Disassembled valve, lapped seat and installed new disc/stem assembly.
- Replaced valve core for Charging Pump CH-1C Suction Accumulator CH-26C due to valve core leakage.
- Replaced valve core for Charging Pump CH-1A Suction Accumulator CH-26A due to valve core leakage.
- Replaced closing springs with one of more reliable design on Breaker 1A4-4 for Heater Drain Pump FW-5C and Breaker 1A4-6 for Condensate Pump FW-2C.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE AUGUST 09, 1994
COMPLETED BY D. L. LIPPY
TELEPHONE 402-533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: JULY 1994

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:

N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	5087.0	182761.0
12. Number of Hours Reactor was Critical	744.0	5053.2	142744.9
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	5038.1	141101.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1112799.6	7428810.4	186714610.0
17. Gross Elec. Energy Generated (MWH)..	365760.0	2493998.0	61582022.2
18. Net Elec. Energy Generated (MWH)....	348529.5	2379508.9	58752571.8
19. Unit Service Factor.....	100.0	99.0	77.2
20. Unit Availability Factor.....	100.0	99.0	77.2
21. Unit Capacity Factor (using MDC Net)	98.0	97.9	69.6
22. Unit Capacity Factor (using DER Net)	98.0	97.9	68.0
23. Unit Forced Outage Rate.....	.0	1.0	4.1

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
NONE

25. If shut down at end of report period, estimated date of startup:

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY N/A
COMMERCIAL OPERATION

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	AUGUST 09, 1994
COMPLETED BY	D. L. LIPPY
TELEPHONE	402-533-6843

MONTH JULY 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	467	17	469
2	467	18	468
3	469	19	465
4	471	20	465
5	470	21	467
6	468	22	468
7	468	23	468
8	468	24	467
9	471	25	465
10	474	26	465
11	471	27	466
12	468	28	469
13	468	29	469
14	470	30	469
15	472	31	469
16	471		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE August 9, 1994
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH July 1994

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
None									During July 1994, the plant operated at a nominal 100% power.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

(9/77)

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending July 31, 1994

1. Scheduled date for next refueling shutdown.	March 11, 1995
2. Scheduled date for restart following refueling.	April 29, 1995
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment?	No
a. If answer is yes, what, in general, will these be?	N/A
b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.	No
c. If no such review has taken place, when is it scheduled?	Prior to April 1995
4. Scheduled date(s) for submitting proposed licensing action and support information.	No submittal planned
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	**
6. The number of fuel assemblies: a) in the core	133 Assemblies
b) in the spent fuel pool	570 Assemblies
c) spent fuel pool storage capacity	729 Assemblies
d) planned spent fuel pool storage capacity	1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1995 Outage*

* Capability of full core offload of 133 assemblies lost. Reracking began in March and is scheduled for completion in August 1994.

** OPPD is utilizing the CASMO-3/SIMULATE-3 codes for reactor physics related analyses for Cycle 16.

Prepared by W O Weber for K.C. Holthaus Date 8/9/94