



Commonwealth Edison
Dresden Nuclear Power Station
6500 North Dresden Road
Morris, Illinois 60450
Telephone 815/942-2920

August 12, 1994

GFSLTR 94-0268

Director, Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, DC 20555

Attention: Document Control Desk

Subject: Monthly Operating Data Report for July 1994
Dresden Nuclear Power Station
Commonwealth Edison Company
Docket Nos. 50-010, 50-237, and 50-249

Gentlemen:

Enclosed is the Dresden Nuclear Power Station Monthly Operating Summary Report for July 1994.

This information is supplied to your office as required by Technical Specification 6.6.A.3, in accordance with the instructions set forth in Regulatory Guide 1.16.

Sincerely,

Gary F. Spedl
Station Manager

GFS/KWS:cah

Enclosure

cc: NRC Region III Office
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MONTHLY NRC
SUMMARY OF OPERATING EXPERIENCE,
PER REGULATORY GUIDE 1.16
FOR
DRESDEN NUCLEAR POWER STATION
COMMONWEALTH EDISON COMPANY
FOR July, 1994

<u>UNIT</u>	<u>DOCKET</u>	<u>LICENSE</u>
1	050-010	DPR-2
2	050-237	DPR-19
3	050-249	DPR-25

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1.0 Introduction

Dresden Nuclear Power Station is a three reactor generating facility owned and operated by the Commonwealth Edison Company of Chicago, Illinois. Dresden Station is located at the confluence of the Kankakee and Des Plaines Rivers, in Grundy County, near Morris, Illinois.

Dresden Unit 1 is a General Electric Boiling Water Reactor with a design net electrical output rating of 200 megawatts electrical (MWe). The unit is retired in place with all nuclear fuel removed from the reactor vessel. Therefore, no Unit 1 operating data is provided in this report.

Dresden Units 2 and 3 are General Electric Boiling Water Reactors with design net electrical output ratings of 794 MWe each.

Waste heat is rejected to a man-made cooling lake using the Kankakee River for make-up and the Illinois River for blowdown.

The Architect-Engineer for Dresden Units 2 and 3 was Sargent and Lundy of Chicago, Illinois.

This report for July, 1994, was compiled by Kevin W. Sykes of the Dresden regulatory Assurance Staff, telephone number (815) 942-2920, extension 2704.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR July, 1994

2.1 UNIT 2 MONTHLY OPERATING EXPERIENCE SUMMARY

07/01/94 Unit 2 entered the month critical and on-line.

Core thermal power has been administratively limited to 99% since 09-24-93 due to the feedwater flow nozzle calibration concern identified at Quad Cities. The nozzle concern is being investigated under Problem Investigation Report (PIR) 12-2-93-117 (NTS #237-200-93-11700).

07/28/94 At 2130 hours a load drop was initiated from approximately 780 MWe to 350 MWe to perform Main Steam Isolation Valve (MSIV) timing and Control Rod Drive (CRD) scram testing in accordance with Dresden Operating Surveillance (DOS) 0250-02, Full Closure Timing and Exercising of Main Steam Isolation Valves, and Dresden Technical Surveillance (DTS) 0300-02, Control Rod Drive Scram Testing and Scram Valve Timing Test, respectively. DOS 0250-02 requires a reduction in power to less than 50% of rated (or as specified by the Operations Shift Supervisor) prior to timing the valves. While the unit was at reduced power, CRD scram solenoid valve diaphragm inspections were conducted under NWRs D26350, D26363 and D26364.

The unit continued on-line through the end of the month.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR July, 1994

2.2 UNIT 3 MONTHLY OPERATING EXPERIENCE SUMMARY

07/01/94 Unit 3 entered the month in refuel and remained shut down through the end of the month.

3.0 OPERATING DATA REPORT

3.1 OPERATING DATA REPORT - DRESDEN UNIT TWO

DOCKET No. 050-237
 DATE August 1, 1994
 COMPLETED BY K. W. Sykes
 TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: **July, 1994**
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
 MAXIMUM DEPENDABLE CAPACITY (MWe NET): 772
 DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe Net): 99% of thermal output (2502 MWth)
4. REASONS FOR RESTRICTIONS (IF ANY): Feedwater flow nozzle calibration concern (see Section 2.1)

REPORTING PERIOD DATA

	PARAMETER	THIS MONTH	YEAR TO DATE	CUMULATIVE
5.	HOURS IN PERIOD	744	5087	211,535
6.	TIME REACTOR CRITICAL (Hours)	744	5022	159,480
7.	TIME REACTOR RESERVE SHUTDOWN (Hours)	0	0	0
8.	TIME GENERATOR ON-LINE (Hours)	744	4995	153,205.6
9.	TIME GENERATOR RESERVE SHUTDOWN (Hours)	0	0	0
10.	THERMAL ENERGY GENERATED (MWh Gross)	1,702,451	11,904,114	317,895,402
11.	ELECTRICAL ENERGY GENERATED (MWe Gross)	523,352	3,740,247	101,351,698
12.	ELECTRICAL ENERGY GENERATED (MWe Net)	497,288	3,572,911	95,832,429
13.	REACTOR SERVICE FACTOR (%)	100	98.7	75.4
14.	REACTOR AVAILABILITY FACTOR (%)	100	98.7	75.4
15.	GENERATOR SERVICE FACTOR (%)	100	98.2	72.4
16.	GENERATOR AVAILABILITY FACTOR (%)	100	98.2	72.4
17.	CAPACITY FACTOR (USING MDC Net) (%)	86.6	91.0	58.7
18.	CAPACITY FACTOR (USING DER Net) (%)	84.2	88.5	57.1
19.	FORCED OUTAGE FACTOR (%)	0	1.8	11.9

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (Type, Date and Duration of Each)
N/A
21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP.
N/A

3.0 OPERATING DATA REPORT

3.2 OPERATING DATA REPORT - DRESDEN UNIT THREE

DOCKET No. 050-249
 DATE August 1, 1994
 COMPLETED BY K. W. Sykes
 TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: **July, 1994**
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
 MAXIMUM DEPENDABLE CAPACITY (MWe Net): 773
 DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe Net): N/A
4. REASONS FOR RESTRICTIONS (IF ANY): N/A

REPORTING PERIOD DATA

5.	HOURS IN PERIOD	744	5087	201,864
6.	TIME REACTOR CRITICAL (Hours)	0	1642	144,348
7.	TIME REACTOR RESERVE SHUTDOWN (Hours)	0	0	0
8.	TIME GENERATOR ON-LINE (Hours)	0	1637	138,952
9.	TIME GENERATOR RESERVE SHUTDOWN (Hours)	0	0	0
10.	THERMAL ENERGY GENERATED (MWh Gross)	0	2,841,650	286,576,467
11.	ELECTRICAL ENERGY GENERATED (MWh Gross)	0	893,683	92,259,719
12.	ELECTRICAL ENERGY GENERATED (MWh Net)	-6694	817,098	87,384,162
13.	REACTOR SERVICE FACTOR (%)	0	32.3	71.5
14.	REACTOR AVAILABILITY FACTOR (%)	0	32.3	71.5
15.	GENERATOR SERVICE FACTOR (%)	0	32.2	68.8
16.	GENERATOR AVAILABILITY FACTOR (%)	0	32.2	68.8
17.	CAPACITY FACTOR (USING MDC Net) (%)	0	20.8	56.0
18.	CAPACITY FACTOR (USING DER Net) (%)	0	20.2	54.5
19.	FORCED OUTAGE FACTOR (%)	0	0	11.6

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (Type, Date and Duration of Each)

Refuel Outage 13, D3R13, began on: March 12, 1994. The outage had originally been scheduled for 13 weeks.

21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP
 August 5, 1994.

3.3 AVERAGE DAILY UNIT 2 POWER LEVEL

DOCKET No. 050-237
 UNIT Dresden 2
 DATE August 1, 1994
 COMPLETED BY K. W. Sykes
 TELEPHONE (815) 942-2920

MONTH: July, 1994

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	702	18	712
2	714	19	727
3	628	20	732
4	659	21	730
5	726	22	722
6	707	23	702
7	714	24	631
8	715	25	688
9	684	26	710
10	659	27	705
11	730	28	695
12	727	29	337
13	713	30	333
14	731	31	399
15	725		
16	720		
17	628		

3.4 AVERAGE DAILY UNIT 3 POWER LEVEL

DOCKET No. 050-249
 UNIT Dresden 3
 DATE August 1, 1994
 COMPLETED BY K. W. Sykes
 TELEPHONE (815) 942-2920

MONTH: July, 1994

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	0	18	0
2	0	19	0
3	0	20	0
4	0	21	0
5	0	22	0
6	0	23	0
7	0	24	0
8	0	25	0
9	0	26	0
10	0	27	0
11	0	28	0
12	0	29	0
13	0	30	0
14	0	31	0
15	0		
16	0		
17	0		

3.5 UNIT 2 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OF July, 1994

NO.	DATE	TYPE(1)	DURATION (HOURS)*	REASON(2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE(4)	COMPO-NENT CODE (5)	CORREC-TIVE ACTIONS/ COM-MENTS
5	940728	S	0	B	5	N/A	N/A	N/A	SEE NOTE 1 BELOW

* Year-to-date forced outage hours = 92.4; Cumulative forced outage hours = 20,712.4

NOTE 1: At 2130 hours a load drop was initiated from approximately 780 MWe to 350 MWe to perform Main Steam Isolation Valve (MSIV) timing and Control Rod Drive (CRD) scram testing in accordance with Dresden Operating Surveillance (DOS) 0250-02, Full Closure Timing and Exercising of Main Steam Isolation Valves, and Dresden Technical Surveillance (DTS) 0300-02, Control Rod Drive Scram Testing and Scram Valve Timing Test, respectively. DOS 0250-02 requires a reduction in power to less than 50% of rated (or as specified by the Operations Shift Supervisor) prior to timing the valves. While the unit was at reduced power, CRD scram solenoid valve diaphragm inspections were conducted under NWRs D26350, D26363 and D26364.

TABLE KEY:

(1)
 F: Forced
 S: Scheduled

(2)
 Reason:
 A Equipment Failure (Explain)
 B Maintenance or Test
 C Refueling
 D Regulatory Restriction
 E Operator Training & Licensing Exam
 F Administrative
 G Operational Error
 H Other (Explain)

4. Other (Explain)
 5. Load Reduction

(4)
 Exhibit G Instruction for Preparation of Data Entry Sheets for Licensee Event Reports (LER) File (NUREG-0161)

(5)
 Exhibit I Same Source as Above.

(3)
 Method:
 1. Manual
 2. Manual Scram
 3. Automatic Scram

3.6 UNIT 3 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1994

NO.	DATE	TYPE(1)	DURATION (HOURS)*	REASON(2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE(4)	COMPO-NENT CODE (5)	CORREC-TIVE ACTIONS/ COM-MENTS
3	940309	S	720	C	1	NA	NA	NA	NA

* Year-to-date forced outage hours = 0; Cumulative forced outage hours = 18,239

TABLE KEY:

- (1)
 F: Forced
 S: Scheduled
- (2)
 Reason:
 A Equipment Failure (Explain)
 B Maintenance or Test
 C Refueling
 D Regulatory Restriction
 E Operator Training & Licensing Exam
 F Administrative
 G Operational Error
 H Other (Explain)

- (3)
 Method:
 1. Manual
 2. Manual Scram
 3. Automatic Scram
 4. Other (Explain)
 5. Load Reduction

(4)
 Exhibit G Instruction for Preparation of Data Entry
 Sheets for Licensee Event Reports (LER) File
 (NUREG-0161)

(5)
 Exhibit I Same Source as Above.

3.7 UNIT 2 MAXIMUM DAILY ELECTRICAL LOAD FOR July, 1994.

Day	Hour Ending	UNIT 2 MAXIMUM DAILY ELECTRICAL LOAD (KWe)
1	1200	777,000
2	1800	780,000
3	2300	782,000
4	1700	778,000
5	1200	777,000
6	2300	765,000
7	0100	766,000
8	1100	774,000
9	1100	777,000
10	1300	782,000
11	1000	784,000
12	1100	776,000
13	1000	776,000
14	1000	777,000
15	1200	776,000
16	1500	776,000
17	1300	776,000
18	0800	777,000
19	1200	776,000
20	1800	777,000
21	1900	775,000
22	0200	774,000
23	0100	766,000
24	1500	744,000
25	1000	761,000
26	2000	777,000
27	2400	779,000
28	0800	781,000
29	0100	528,000
30	1400	372,000
31	2400	712,000

3.8 UNIT 3 MAXIMUM DAILY ELECTRICAL LOAD FOR July, 1994

Day	Hour Ending	UNIT 3 MAXIMUM DAILY ELECTRICAL LOAD (KWe)
1		0
2		0
3		0
4		0
5		0
6		0
7		0
8		0
9		0
10		0
11		0
12		0
13		0
14		0
15		0
16		0
17		0
18		0
19		0
20		0
21		0
22		0
23		0
24		0
25		0
26		0
27		0
28		0
29		0
30		0
31		0

4.0 UNIQUE REPORTING REQUIREMENTS

4.1 MAIN STEAM RELIEF VALVE OPERATIONS

None.

4.2 OFF-SITE DOSE CALCULATION MANUAL (ODCM) CHANGES

None.

4.3 MAJOR CHANGES TO THE RADIOACTIVE WASTE TREATMENT SYSTEMS
July, 1994

None.

4.4 FAILED FUEL ELEMENT INDICATIONS

4.4.1 Unit 2

Unit 2 fuel performance during July, 1994, continued to show no indications of leaking fuel. This is based on the sum of the activities of the six (6) Noble Gases as measured at the Recombiner. Therefore, Unit 2 had excellent fuel performance.

4.4.2 Unit 3

Unit 3 was shut down during the month of July for Refuel Outage D3R13.

5.0 TECHNICAL SPECIFICATION AMENDMENTS AND SAFETY RELATED MAINTENANCE

5.1 Amendments to Facility License or Technical Specifications implemented during July, 1994.

(1) The NRC issued Technical Specification amendments 128 (DPR-19, Unit 2) and 122 (DPR-25, Unit 3). Amendment 122 was implemented on 07/19/94. Amendment 128 to DPR-19 will be distributed upon operational authorization of the RVWLIS backfill system after its installation.

These amendments revise the Limiting Conditions for Operation (LCOs) and Surveillance Requirements (SRs) for Primary Containment Isolation Valves to reflect the addition of the RVWLIS reference leg backfill system check valves as part of the reactor coolant pressure boundary and primary containment.

The notable changes to the technical specifications concerning the RVWLIS amendment are:

- a) The requirement to have the RVWLIS reference leg backfill check valves operable whenever primary containment integrity is required has been added (3.7.D.1).
- b) The requirement to close a valve in the backfill line that has an inoperable backfill check valve has been added (3.7.D.2)
- c) A Surveillance Requirement has been added to record the position of the closed valve (from 3.7.D.2) daily.

The provisions of these amendments are effective immediately for Unit 3 and for Unit 2 upon operational authorization of the RVWLIS backfill system after its installation.

(2) The NRC issued Technical Specification amendments 127 (DPR-19, Unit 2) and 121 (DPR-25, Unit 3). These amendments were implemented on 07/19/94.

These amendments revise the Limiting Conditions for Operation (LCOs) and Surveillance Requirements (SRs) for continued operation with one reactor recirculation loop in service.

The notable changes to the technical specifications concerning the SLO amendments are:

- a) The requirement to close the idle recirculation pump suction valve has been deleted (3.6.H.3.e).

- b) LCO's and Surveillance Requirements (SR's) for continuous operation within the region of instability have been deleted. (3.6.H.3.a - d, 4.6.H.3.a and b).
- c) Figure 3.6-2 (region of instability) has been deleted. This figure is not required since continued operation within the region of instability is no longer allowed. The region of instability will continued to be defined as the region of power operation where the Flow Control Line is greater than 80 percent and total core flow is less that 45 percent. (DOS 0500-18).
- d) Operation without forced recirculation is no longer allowed. (3.6.H.4)
- e) New Sections 3.6.H.5 a nd 4.6.H.5 contain LCO's and SR's that assure that a startup of the idle recirculation loop is performed safely.

5.2 Safety Related Maintenance (Unit 2 and 3)

5.2.1 Safety related preventive maintenance activities for July, 1994 are summarized in the attached computer printout. Unit 2/3 items are designated as "00" in the "Unit" column.

SELECTION CRITERIA: MAINT COMP DATE IN RANGE OF '07/01/94' THRU '07/31/94' NUC SAFE EQ 'Y' JOB CODE STARTS WITH 'P'

REQ	UNIT	PN	STD DESCRIPTION	MAINT COMP DATE	WORK REQ
1	03	3-260-287	VALVE CHECK SPARE FEEDWATER #007	07/09/94	D26507
1	03	3-3-7510-3	SOLENOID SHUT EXH FAN SUCTION ULV	07/06/94	D26353
1	03	3-309-39-11	HYDRAULIC CONTROL UNIT #30-11	07/24/94	D26770
4	03	3-1010	SITE #11 BUILDING AND GROUNDS MAINTENANCE	07/19/94	D25410
5	03	3-1010	SITE #10 BUILDING AND GROUNDS MAINTENANCE	07/19/94	D25411
6	03	3-1200	MISC SYSTEM NUCLEAR BOILER(RECIRC)	07/05/94	D26056
7	03	3-1200	MISC SYSTEM NUCLEAR BOILER(RECIRC)	07/05/94	D26057
8	03	3-1203-14	VALVE IN INBOARD MAIN STEAM ISOLATION 14	07/13/94	D12807
9	03	3-1203-10	VALVE IN INBOARD MAIN STEAM ISOLATION 10	07/07/94	D12868
10	03	3-1203-24	OPERATOR ADV OUTBOARD MSIV 24	07/14/94	D22779
11	03	3-1203-28	OPERATOR ADV OUTBOARD MSIV 28	07/15/94	D22778
12	03	3-1203-20	OPERATOR ADV OUTBOARD MSIV 20	07/15/94	D22777
13	03	3-1220-44	SOLENOID ADV RECIRC LOOP SAMPLE ISO 3-1220-44	07/25/94	D26791
14	03	3-1220-52	OPERATOR RM HEAD O-RING LEAKOFF ADV 220-52	07/25/94	D26792
15	03	3-1220-52	SOLENOID RM HEAD O RING LEAKOFF ADV 3-1220-52	07/25/94	D26792
16	03	3-1262-25A	VALVE FLOW LIMITING CHECK PT 262-7C	07/20/94	D26565
17	03	3-163-113	TRANSMITTER WIDE RANGE REACTOR LEVEL	07/12/94	D92860
18	03	3-700	ELEMENT LPRM LOCAL POWER RANGE MONITOR	07/13/94	D04719
19	03	3-8700-06-17C	AMPLIFIER CARD LOCAL POWER RANGE MONITOR	07/18/94	D26693
20	03	3-0719-24-17	DETECTOR NEUTRON 4 ELEMENT STRING LPRM 24-17	07/23/94	D04719
21	03	3-0719-24-25	DETECTOR NEUTRON 4 ELEMENT STRING LPRM 24-25	07/23/94	D04719
22	03	3-0719-32-49	DETECTOR NEUTRON 4 ELEMENT STRING LPRM 32-49	07/23/94	D04719

OBS

DESCRIPTION

1 REQUESTED--- UPGRADE TO NEW DESIGN.-----

2 REQUESTED--- AIR LEAK JUST ABOVE EXHAUST PORT ON SOLENOID ULV 2/3-7541-43B. REPAIR + REPLACE

3 REQUESTED--- THREADS ON W2 BLOCK FOR ACCUMULATOR 30-11 (N-3) NEED TO BE RETRIPPED, W2 LEAK

4 REQUESTED--- SEAL 8" PENETRATION @ EL 434'-0" BEHIND BASEPLATE ON TORUS WALL SIDE-----

5 REQUESTED--- SEAL 20" PENETRATION @ EL 485'-0" 7'-3" ON TORUS WALL SIDE.-----

6 REQUESTED--- INSTALL HT PRESS MECHAN JUMPER FOR VESSEL HYDRO. BETWEEN TAPS 3-299-12A + 113A--

7 REQUESTED--- INSTALL HT PRESS MECHAN JUMPER FOR VESSEL HYDROBETWEEN TAPS 3-299-112B + 113B----

8 REQUESTED--- CLEAN+LUBRICATE STAIN AND GUIDE POSTS.-----

9 REQUESTED--- REMOVE+ADD+REPAIR+PAINT CLEAN+LUBRICATE STEM + GUIDE POSTS.-----

10 REQUESTED--- REMOVE, INSPECT, INSTALL ADDITIONAL PARTS ON DOUBLE SEAL ASSEMBLY + MAKE REPAIRS

11 REQUESTED--- REMOVE, INSPECT, INSTALL ADDITIONAL PARTS ON DOUBLE SEAL ASSEMBLY MAKE REPAIRS--

12 REQUESTED--- REMOVE, INSPECT, INSTALL ADDITIONAL PARTS ON THE DOUBLE SEAL ASSEMBLY-----

13 REQUESTED--- REPLACE SOLENOID OR REBUILD AS REQUIRED.-----

14 REQUESTED--- REPLACE SOLENOID OR REBUILD AS REQUIRED.-----

15 REQUESTED--- REPLACE SOLENOID OR REBUILD AS REQUIRED.-----

16 REQUESTED--- BENCH TEST INST FLOW CHECK #013 0500-049 REQ BY TS-----

17 REQUESTED--- REMOVE 8" RW ISOLATION VALVE FOR L73-263-113-----

18 REQUESTED--- REPLACE 03 LPRMS THAT HAVE REACHED THEIR ELECT OR NUC END OF LIFE-----

19 REQUESTED--- HIGH ALARM CAME UP AT 70 WATT/CM. FOR 009 700-9 ALARM SETPT SHOULD BE < 75.5 WATT

20 REQUESTED--- REPLACE 03 LPRMS THAT HAVE REACHED THEIR ELECT OR NUC END OF LIFE-----

21 REQUESTED--- REPLACE 03 LPRMS THAT HAVE REACHED THEIR ELECT OR NUC END OF LIFE-----

22 REQUESTED--- REPLACE 03 LPRMS THAT HAVE REACHED THEIR ELECT OR NUC END OF LIFE-----

SELECTION CRITERIA: MAINT COMP DATE IN RANGE OF '07/01/94' THRU '07/31/94' NUC SAFE EQ 'V' JOB CODE STARTS WITH 'P'

OBS	WT	PN	EIT DESCRIPTION	MAINT COMP DATE	WORK REQ
23	03	3-1601-40-09	DETECTOR NEUTRON 4 ELEMENT STRING LPRM 40-09	07/23/94	D04719
24	03	3-1601-40-49	DETECTOR NEUTRON 4 ELEMENT STRING LPRM 40-49	07/23/94	D04719
25	03	3-1601-12-17A	AMPLIFIER CARD LOCAL POWER RANGE MONITOR	07/15/94	D26685
26	03	3-1601-1	VALVE NO ISOLATION CONDENSER RETURN ISOLATION	07/06/94	D02630
27	03	3-1601-22B	OPERATOR MOV 1501-22B	07/22/94	D26702
28	03	3-1601-44A	MOTOR A CONTAINMENT COOLING SERV WATER PUMP	07/12/94	D26531
29	03	3-1602-0	MOTOR C LPCI PUMP	07/15/94	D12115
30	03	3-1600	MISC SYSTEM PRESSURE SUPPRESSION	07/13/94	D26556
31	03	3-1600	MISC SYSTEM PRESSURE SUPPRESSION	07/11/94	D26557
32	03	3-1601-20B	OPERATOR AD VALVE DRYWELL VENT #20B	07/14/94	D13181
33	03	3-1601-23	ACCUMULATOR AD VALVE DRYWELL VENT #23	07/16/94	D13161
34	03	3-1601-23-2	VALVE CHECK AIR SUPPLY FROM ACCUM 3-1601-23	07/14/94	D13162
35	03	3-1601-24	OPERATOR MOV 3-1601-24	07/13/94	D13164
36	03	3-1601-24-1	VALVE CHECK AIR SUPPLY FROM ACCUM 3-1601-24	07/14/94	D13166
37	03	3-1601-24-2	VALVE CHECK AIR SUPPLY FROM ACCUM 3-1601-24	07/20/94	D13166
38	03	3-1601-56	OPERATOR AD VALVE DRYWELL VENT #56	07/15/94	D13167
39	03	3-1601-58	OPERATOR AD VALVE DRYWELL VENT #58	07/12/94	D12816
40	03	3-1601-59	OPERATOR AD VALVE DRYWELL VENT #59	07/12/94	D12817
41	03	3-1601-60	OPERATOR AD VALVE DRYWELL VENT #60	07/18/94	D13170
42	03	3-1601-60-1	VALVE CHECK AIR SUPPLY FROM ACCUM 3-1601-60	07/08/94	D13170
43	03	3-1601-60-2	VALVE CHECK AIR SUPPLY FROM ACCUM 3-1601-60	07/08/94	D13170
44	03	3-1601-61	OPERATOR AD VALVE DRYWELL VENT BYPASS	07/20/94	D12819

OBS	DESCRIPTION
23	REQUESTED--- REPLACE US LPRMS THAT HAVE REACHED THEIR ELECT OR NUC END OF LIFE-----
24	REQUESTED--- REPLACE US LPRMS THAT HAVE REACHED THEIR ELCT OR NUC END OF LIFE-----
25	REQUESTED--- HIGH ALARM CAME UP AT 105 WATT/CM. PER DOS 0700-9 ALARM SET POINT 90.5 WATT/CM--
26	REQUESTED--- REPACK VALVE-----
27	REQUESTED--- REPLACE STEM NUT, CLEAN STEM AND REGREASE TO IMPROVE ULU STEM FACTOR-----
28	REQUESTED--- PUMP TRIPPED ON THIS DELAYED OVERCURRENT AFTER BEING SLATED FOR RUN-----
29	REQUESTED--- OVERHAUL 30 LPCI MOTOR DURING 03R10 OUTAGE.-----
30	REQUESTED--- INSTALL BLIND FLANGE AT PENETRATION 4318 TO SUPPORT LLRT.-----
31	REQUESTED--- INSTALL BLIND FLANGE AT PENETRATION 4326 TO SUPPORT LLRT.-----
32	REQUESTED--- INSPECT REPAIR AIR OPERATOR HOSE REPLACEMENTSEE WR 12752.-----
33	REQUESTED--- INSPECT REPAIR AIR OPERATOR + REPLACE AIR HOSES.-----
34	REQUESTED--- REPLACE CHECK VALVE.-----
35	REQUESTED--- INSPECT REPAIR AIR OPERATOR + REPLACE AIR HOSES.-----
36	REQUESTED--- REPLACE CHECK VALVE.-----
37	REQUESTED--- REPLACE CHECK VALVE.-----
38	REQUESTED--- INSPECT REPAIR AIR OPERATOR + REPLACE AIR HOSES.-----
39	REQUESTED--- INSPECT OPERATOR REPLACE DIAPHRAGM.-----
40	REQUESTED--- INSPECT OPERATOR REPLACE DIAPHRAGM.-----
41	REQUESTED--- INSPECT REPAIR AIR OPERATOR + REPLACE AIR HOSES.-----
42	REQUESTED--- INSPECT REPAIR AIR OPERATOR + REPLACE AIR HOSES.-----
43	REQUESTED--- INSPECT REPAIR AIR OPERATOR + REPLACE AIR HOSES.-----
44	REQUESTED--- INSPECT OPERATOR REPLACE DIAPHRAGM.-----

SELECTION CRITERIA: MAINT COMP DATE IN RANGE OF '07/01/94' THRU '07/31/94' NUC SAFE EQ 'Y' JOB CODE STARTS WITH 'P'

DBF	UNIT	EPN	EID DESCRIPTION	MAINT COMP DATE	WORK REQ
45	01	3-1601-62	OPERATOR RC VALVE DRYWELL VENT BYPASS #62	07/13/94	D12818
46	02	3-1641001-34	POWER SUPPLY MAIN DRYWELL AND TORUS LEVEL PNL	07/09/94	D26391
47	02	3-2540-164	SWITCH CONTAINMENT ATMOSPHERE PRESSURE	07/20/94	D26733
48	02	3-2540-166	SWITCH CONTAINMENT ATMOSPHERE PRESSURE	07/20/94	D26732
49	02	3-2540-174	SWITCH CONTAINMENT ATMOSPHERE PRESSURE	07/20/94	D26733
50	02	3-2540-194	SWITCH CONTAINMENT ATMOSPHERE PRESSURE	07/20/94	D26733
51	02	3-2540-194	SWITCH CONTAINMENT ATMOSPHERE PRESSURE	07/20/94	D26733
52	02	3-4327-501	CLEAN DEMIN WTR SU	07/08/94	D26497
53	02	3-4327-502	CLEAN DEMIN WTR SU TO SERVICE STATION	07/08/94	D26498
54	02	3-5700-30A	COOLER CDSW PUMP CUBICLE	07/25/94	D26475
55	03	3-6728-71001	SWITCHGEAR BREAKER 4KV.	07/23/94	D19384
56	03	3-6733-09	BUS CUBICLE FEED SD CLG PF 3A-1002	07/06/94	D24235
57	03	3-6734-33	BUS BREAKER FEED SD CLG PF 3C-1002	07/06/94	D24148
58	03	3-7838-28	SWGR BRK MAIN FEED FROM 3-6733-25	07/01/94	D19102
59	03	3-7839-30	SWGR BRK TB MCC 39-2 FEED 3-7839-2A6	07/06/94	D26354
60	03	3-7838-302	SWGR BRK CDSW PF CUB COOLER FAN#2 3-5700-30A	07/03/94	D19163
61	03	3-7839-141	SWGR BRK HPCI AUX COOLANT WTR PF 3-2301-57	07/24/94	D19162
62	03	3-7839-181	SWGR BRK HPCI PF 3 AREA CLG UNIT 3-5747	07/13/94	D19161
63	03	3-7839-1F3	SWGR BRK HPCI OIL TANK HEATER	07/05/94	D19160

DBF

DESCRIPTION

45	REQUESTED---	INSPECT OPERATOR/REPLACE DIAPHRAGM.-----
46	REQUESTED---	INVESTIGATING THE REASON FOR OUTPUT FROM I/E 3-1640-105 MODL POWER IS BAD-----
47	REQUESTED---	DIS 2500-3 ACAD LOGIC TEST THE SETPOINTS WEREMANIPULATED ON SWITCHES 3-2540 16A-
48	REQUESTED---	DIS 2500-3 ACAD LOGIC TEST THE SETPOINTS WEREMANIPULATED ON SWITCHES 3-2540-16B-
49	REQUESTED---	DIS 2500-3 ACAD LOGIC TEST THE SETPOINTS WEREMANIPULATED ON SWITCHED 3-2540 16A-
50	REQUESTED---	DIS 2500-3 ACAD LOGIC TEST THE SETPOINTS WEREMANIPULATED ON SWITCHES 3-2540 16A-
51	REQUESTED---	DIS 2500-3 ACAD LOGIC TEST THE SETPOINTS WEREMANIPULATED ON SWITCHES 3-2540 16A-
52	REQUESTED---	VALVE LEAKED @ 30 PSI DURING AS FOUND LLRT. PLEASE REPAIR-----
53	REQUESTED---	VALVE LEAKED ABOUT 30 PSID DURING AS-FOUND LLRT. PLEASE REPAIR-----
54	REQUESTED---	UPPER ROOM COOLER FOR THE US 'B' CDSW PMP IS LEAKING. REPAIR-----
55	REQUESTED---	PERFORM REFUEL BY SWGR OF BRK + CUBICLE ASSIGNED TO BUSES 33/34.-----
56	REQUESTED---	OBTAIN MOTOR STARTING CURVE WITH PUMP COUPLED WITH A CHART RECORDER-----
57	REQUESTED---	A MOTOR STARTING CURVE NEEDS TO BE DEVELOPED FOR 3C SHUTDOWN COOLING PUMP BECASU
58	REQUESTED---	PERFORM REFUEL BY SWGR OF BRK + CUBICLE AT BUS 38-----
59	REQUESTED---	UPGRADE BREAKER WITH R15-A DEVICE FOR BUS 39 FEED TO MCC 39-2-----
60	REQUESTED---	PERFORM REFUEL BY SWGR OF BRK + CUBICLE AT MCC 38-3-----
61	REQUESTED---	PERFORM REFUEL BY SWGR OF BRK + CUBICLE AT MCC 39-1-----
62	REQUESTED---	PERFORM REFUEL BY SWGR OF BRK + CUBICLE AT MCC 39-1-----
63	REQUESTED---	PERFORM REFUEL BY SWGR OF BRK + CUBICLE AT MCC 39-1-----

5.2 Safety Related Maintenance (Unit 2 and 3)

5.2.2 Safety related corrective maintenance activities for July, 1994 are summarized in the attached computer printout. Unit 2/3 items are designated as "00" in the "Unit" column.

*-SELECTION CRITERIA: MAINT COMP DATE IN RANGE OF '07/01/94' THRU '07/31/94' NUC SAFE EQ 'Y' JOB CODE STARTS WITH 'C'

CDB	JOB	REQ	EEO DESCRIPTION	MAINT	
				COMP DATE	WORK REQ
1	01	3-1794-076	CONTROLLER W BOOT SYSTEM FLOW	07/13/94	D25886
2	01	3-981	CRD SYSTEM 02 015	07/30/94	D26350
3	01	3-382	CRD SYSTEM 02 015	07/29/94	D26363
4	01	3-802	CRD SYSTEM 02 015	07/30/94	D26364
5	02	3-1705-148	TRIP UNIT PERCEL FLOOR (1) B AREA RAD IND	07/01/94	D26503
6	02	3-3989-88	VALVE B/L PAN D/B COOLING WATER SUPPLY DRAIN	07/21/94	D23057
7	02	3-721-148	SWITCHING 4 FEEDER W/IN	07/06/94	D26509
8	02	3-3300-0	BATTERY CHARGER 120V 0	07/09/94	D26579
9	02	3-4200-14	OPERATOR MCH INBOARD MCH 0A	07/20/94	D26701
10	02	3-4260-140B	TRIP/MULTIPLIER UNIT AX LOW LEVEL SCRAM/ISOLATION	07/12/94	D26660
11	02	3-9300	MISC SYSTEM CONTROL ROD DRIVE HYDRAULICS	07/30/94	D26830
12	02	3-9700	MISC SYSTEM NEUTRON MONITORING	07/16/94	D26691
13	02	3-1301-1	OPERATOR MCH 1301-1	07/23/94	D26727
14	02	3-1301-17	VALVE AD F/W 100 CONDENSER EXHAUST VENT	07/16/94	D21317
15	02	3-1402-388	VALVE MCH CORE SPRAY RECIRCULATION	07/28/94	D26780
16	02	3-1501-378	MOTOR MOV 1501-378	07/24/94	D26783
17	02	3-1601-12	OPERATOR MCH 3-1601-12	07/03/94	D26558
18	02	3-1640-110	ISOLATOR CRAWELL PRESS PRIM COMT LEVEL SIGNAL	07/07/94	D26571
19	02	3-1900	SYSTEM FUEL POOL PIPING WELDS NON-CLASS	07/12/94	D26473
20	02	3-7800-39-1	MOTOR CONTROL CENTER 39-1	07/23/94	D13916

086

DESCRIPTION

1 REQUESTED--- REPAIR INSTRUMENT AIR SUPPLY LINE FOR STANDBYGAS TREATMENT 'B' TRAIN CONTROLLER.
 2 REQUESTED--- INSPECT RUBBER DIAPHRAGMS AND REPLACE AS NECESSARY OR MCH'S FOR CRD'S C-8,C-11,E-
 3 REQUESTED--- INSPECT RUBBER DIAPHRAGMS + REPLACE ON MCH FORCRD'S C-7,E-6,F-8,J-8, J-11, K-5---
 4 REQUESTED--- INSPECT RUBBER DIAPHRAGMS + REPLACE ON MCH FORCRD B-7,E-7,O-3,H-12, J-6,J-9, K-13
 5 REQUESTED--- DETECTOR PEGGED GREATER THAN 100 MPH/HR REPAIR AS NEEDED-----
 6 REQUESTED--- MCH LEAK AT STEM. THIS MCH IS ONE OF THE HYPOCHLORITE INJECTION POINTS FOR CHEM
 7 REQUESTED--- REPAIR CONTACTS ON RIGHT SIDE OF BREAKER (BRK# 0204A2316-011)-----
 8 REQUESTED--- UD 120V BATT CHARGER VOLTAGE SWINGING FROM 130V TO 121V. INVESTIGATE + REPAIR---
 9 REQUESTED--- REPAIR MCH AIR LEAK ON THE AIR SUPPLY LINES + MCH OPERATOR AS REQUEST-----
 10 REQUESTED--- TRIP ADJUSTMENT ON 75 SEPARATE AND COULD NOT BE ADJUSTED.-----
 11 REQUESTED--- IDENTIFY AND REPLACE THE BOLT.-----
 12 REQUESTED--- LAMP MCH B DOWNSIDE TRIPPING LAMP BACK PANEL WOULD NOT LIFT. FRONT PANEL DID
 13 REQUESTED--- CYCLED THE VALVE 0.500 OPEN THEN CLOSE. WHEN THE MCH CLOSED THE BKR TRIPPED--
 14 REQUESTED--- AD 1-100-17 MCH POINTING TO LEAKING. PLEASE REPAIR.-----
 15 REQUESTED--- MCH DID NOT OPEN WHEN REQUIRED. THIS IS A MINFLOW MCH + OPENS ON LO FLOW-----
 16 REQUESTED--- VALVE TRIPPING THEMATICALLY-----
 17 REQUESTED--- REPLACE LEAKING O-RING VALVE ON 01611-22 VALVE.-----
 18 REQUESTED--- WHILE REPAIRING ON 01611-22 VALVE, ISOLATOR 3-1640-110 WAS FOUND OUT OF TOLER
 19 REQUESTED--- REPAIR AD FOR 100-17 MCH POINTING TO LEAKING-----
 20 REQUESTED--- TRANSFORMER IS OVERHEATING. PLEASE INVESTIGATE AND REPAIR.-----