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Docket No. 50-289

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Mr. Henry D. Hukill
Vice President
GPU Nuclear Corporation
P. O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Hukill:

Attached are two draft tables reflecting our understanding of the status of certain items at TMI-1. We propose a meeting with you on June 14, 1983 to discuss the status of the listed items. Your NRC Project Manager will contact you shortly to arrange such a meeting. Thank you for your cooperation.

Sincerely,

ORIGINAL SIGNED BY:

JOHN F. STOLZ

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:
As Stated

cc w/enclosures:
See next page

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PDR

OFFICE	ORB#4:DL	C-ORB#4:DL				
SURNAME	JVan Vliet	cf JStolz				
DATE	6/9/83	6/17/83				

Mr. R. J. Togle
Manager, TMI-1
GPU Nuclear Corporation
P. O. Box 480
Middletown, Pennsylvania 17057

Board of Directors
P. A. N. E.
P. O. Box 268
Middletown, Pennsylvania 17057

*Docketing and Service Section
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Chauncey Kepford
Judith H. Johnsrud
Environmental Coalition on Nuclear Power
433 Orlando Avenue
State College, Pennsylvania 16801

*Judge Reginald L. Gotchy
Atomic Safety & Licensing Appeal Board
U.S. Nuclear Regulatory Commission
Washington, DC 20555

J. B. Lieberman, Esq.
Berlock, Israel & Lieberman
26 Broadway
New York, New York 10004

Regional Administrator
U. S. N. R. C., Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

ANGRY/TMI PIRC
1037 Maclay Street
Harrisburg, Pennsylvania 17103

John Levin, Esq.
Pennsylvania Public Utilities
Commission
Box 3265
Harrisburg, Pennsylvania 17120

Jordan D. Cunningham, Esq.
Fox, Farr and Cunningham
2320 North 2nd Street
Harrisburg, Pennsylvania 17110

Ms. Louise Bradford
TMIA
1011 Green Street
Harrisburg, Pennsylvania 17102

Ms. Marjorie M. Aamodt
R. D. #5
Coatesville, Pennsylvania 19320

Earl B. Hoffman
Dauphin County Commissioner
Dauphin County Courthouse
Front and Market Streets
Harrisburg, Pennsylvania 17101

Union of Concerned Scientists
c/o - Harmon & Weiss
1725 I Street, N. W.
Suite 506
Washington, D. C. 20006

Mr. Steven C. Sholly
Union of Concerned Scientists
1346 Connecticut Avenue, N. W.
Dupont Circle Building, Suite 1101
Washington, D. C. 20036

*Gary J. Edles, Chairman
Atomic Safety & Licensing Appeal
Board
U.S. Nuclear Regulatory Commission
Washington, DC 20555

*Dr. John H. Buck
Atomic Safety & Licensing Appeal
Board
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120

General Counsel
Federal Emergency Management Agency
ATTN: Docket Clerk
1725 I Street, NW
Washington, DC 20472

Karin W. Carter, Esq.
505 Executive House
P. O. Box 2357
Harrisburg, Pennsylvania 17120

G. F. Trowbridge, Esq.
Shaw, Pittman, Potts & Trowbridge
1800 M Street, N.W.
Washington, D. C. 20036

Dauphin County Office Emergency
Preparedness
Court House, Room 7
Front & Market Streets
Harrisburg, Pennsylvania 17101

Mr. E. G. Wallace
Licensing Manager
GPU Nuclear Corporation
100 Interpace Parkway
Parsippany, New Jersey 07054

William S. Jordan, III, Esq.
Harmon & Weiss
1725 I Street, NW, Suite 506
Washington, DC 20006

Ms. Lennie Prough
U. S. N. R. C. - TMI Site
P. O. Box 311
Middletown, Pennsylvania 17057

Ms. Virginia Southard, Chairman
Citizens for a Safe Environment
264 Walton Street
Lemoyne, Pennsylvania 17043

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Mr. David D. Maxwell, Chairman
Board of Supervisors
Londonderry Township
RFD#1 - Geyers Church Road
Middletown, Pennsylvania 17057

Mr. C. W. Smyth
Supervisor of Licensing TMI-1
GPU Nuclear Corporation
P. O. Box 480
Middletown, Pennsylvania 17057

Regional Radiation Representative
EPA Region III
Curtis Building (Sixth Floor)
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

Mr. Richard Conte
Senior Resident Inspector (TMI-1)
U.S.N.R.C.
P. O. Box 311
Middletown, Pennsylvania 17057

Governor's Office of State Planning
and Development
ATTN: Coordinator, Pennsylvania
State Clearinghouse
P. O. Box 1323
Harrisburg, Pennsylvania 17120

TABLE 1

 RESTART CERTIFICATION ITEMS

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
	<u>Commission Order of 8/9/79</u>				
	<u>EFW Reliability</u>				
1	Auto Initiation of EFW	Provide automatic initiation of the motor driven EFW pumps on loss of both feedwater pumps or loss of four RCPs. (See also items 81 and 121).	NUREG-0680 Inspection Report (IR-80-26) IR-82-01 IR-81-28 IR-82-26	Complete. Test during power escalation testing (PET) to be implemented by license condition.	No (Reg. 1) PET (9/83)
2	EFW Valves Fail Open	Modify the EFW control valves such that they fail open on loss of air.	NUREG-0680 IR-81-22 IR-82-01 IR-80-26 IR-81-28 IR-82-06	Complete	Yes
3	Auto EFW Load on Diesels	Provide automatic block loading of the motor driven EFW pumps on the diesel.	NUREG-0680 IR-81-14 IR-80-26 IR-82-01 IR-81-28 IR-82-26 IR-83-14	Complete.	Yes
4	EFW Tech Specs	Incorporate EFW in the IMI-1 Technical Specifications per IE Bulletin 79-05A, Item 8. Staff verify that EFW capacity Technical Specification requirements are in accordance with accident analyses.	License Amendment (AMD) 78. IR-81-17 IR-81-21 IR-83-14	Complete.	Yes

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
5	EFW Flow Indication (Control Grade)	Provide indication in the control room of EFW flow to each steam generator. (See also item 82).	NUREG-0680 IR-82-17 IR-83-01 IR-83-02 IR-83-06	Licensee installing safety grade system per long-term requirement. Installed modification unsatisfactory, replacement modification required. Estimated review completion 8/83 (NRR).	No (Reg. 1) 8/83
6	EFW Procedures	Provide procedures and training to assure that EFW is available and properly applied when required. Procedures will identify the need to verify proper operation when EFW is initiated.	NUREG-0680	Complete	N/A
7	EFW Alignment	To assure that EFW will be aligned in a timely manner to inject on all EFW demand events when in surveillance test mode, procedures will be implemented and training conducted to provide an operator at the necessary location in communications with the control room during the surveillance mode to carry out alignment changes necessary upon EFW demand events.	NUREG-0680 IR-81-21	Complete	Yes
8	EFW Auto-Start Annunciation	Design review and modifications, as necessary, will be conducted to provide control room annunciation for all auto start conditions of the EFW system.	NUREG-0680 IR-81-28 IR-82-01	Complete	Yes
<u>Reliability Analysis</u>					
9	CSI Level Alarm	Provide redundant level indication and control room alarms for the primary EFW system water supply.	NUREG-0680 IR-82-01 IR-82-19 IR-82-26	Complete	Yes

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
10	Endurance Test	Perform a 48-hour endurance test on all EFW system pumps.	IR-81-28	Complete	Yes
11	Transfer of EFW Supply	Revise emergency procedures to assure proper transfer of EFW supply to alternate sources when required.	NUREG-0680	Complete	N/A
12	EFW to Intact DTSG	Provide automatic termination of EFW flow to a depressurized steam generator and automatic supply to the intact steam generator.	NUREG-0680	Complete	N/A
13	Auto EFW Protection on Loss of Water Source	Evaluate the need for automatic protection of EFW system pumps on loss of unprotected primary EFW system water source (following a seismic event or tornado).	NUREG-0680	Complete	N/A
14	EFW Initiation Independent of AC	Verify that EFW system initiation and operation is assured independent of alternating current power source for at least two hours.	NUREG-0680 IR-82-01	Complete. Test during PET to be implemented by license condition.	No (Reg. 1) PET (9/83)
15	Cross-Tie Break	Perform volumetric nondestructive examination (NDE) of 10 welds in the EFW pump discharge lines.	IR-81-26 IR-82-19	Complete	Yes
16	EFW Independent of ICS	Develop and implement operating procedures for initiating and controlling EFW independent of Integrated Control System (ICS) control.	NUREG-0680 IR-81-22 IR-81-28 IR-82-03	Complete	Yes
17	Small Break Analysis	Complete analyses for potential small breaks and develop and implement operating instructions to define operator action.	NUREG-0680 IR-83-11	Complete	Yes

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
18	Operator Retraining	Augment the retraining of all Reactor Operators and Senior Reactor Operators assigned to the control room, including training in the areas of natural circulation and small break loss-of-coolant accidents, including revised procedures and the TMI-2 accident. All operators will also receive training at the B&W simulator on the TMI-2 accident and the licensee will conduct a 100 percent reexamination of all operators in these areas. NRC will administer complete examinations to all licensed personnel in accordance with 10 CFR 55.20-23.	NUREG-0680	Complete	N/A
	<u>IE Bulletins</u>				
19	Accident Understanding	Review TMI-2 accident circumstances and chronology per IE Bulletin 79-05A.	NUREG-0680	Complete	N/A
20	Plant Transient Review	Review past transients and provide details of significant deviations from expected performance per IE Bulletin 79-05A.	IR-83-02	Complete	Yes
21	Transient Procedures	Review procedures for coping with transients and accidents per IE Bulletin 79-05A.	NUREG-0680	Complete	N/A
22	Operating Procedures	Review procedures to ensure operators do not override automatic ESF initiation per IE Bulletin 79-05A.	NUREG-0680	Complete	N/A
23	Valve Position Review	Perform a complete safety-related system valve lineup prior to plant restart. (See also item 116).	--	Perform verification at HFT (8/83)	No (Reg. 1) 9/83

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
24	EFW Valve Procedures	Prepare and implement procedures to assure critical EFW valves remain in the correct position.	NUREG-0680	Complete	N/A
25	EFW Operability	Prepare and implement procedures to assure operability of two 100% capacity EFW flow paths per IE Bulletin 79-05A.	NUREG-0680	Complete	N/A
26	Transfer of Containment Liquids	Review systems and procedures to assure that undesired inadvertent pumping of radioactive liquids and gases will not occur per IE Bulletin 79-05A.	NUREG-0680	Complete	N/A
27	Safety System Operability	Review and modify procedures to verify operability of safety systems prior to removing redundant systems from service per IE Bulletin 79-05A.	NUREG-0680	Complete	N/A
28	Personnel Actions TMI-2	Advise operating and maintenance personnel of the extreme seriousness and consequences of the simultaneous blocking of both auxiliary feedwater trains at the Three Mile Island Unit 2 plant and other actions taken during the early phases of the accident.	NUREG-0680	Complete	N/A
29	Prompt Reporting	Review prompt reporting procedures for NRC notification to assure very early notification of serious events.	NUREG-0680	Complete	N/A
30	Natural Circulation	Develop procedures and train personnel on methods of establishing and maintaining natural circulation per IE Bulletin 79-05B.	NUREG-0680	Complete	N/A
31	Vessel Integrity	Modify procedures of Item 22 above to consider vessel integrity per IE Bulletin 79-05B.	NUREG-0680	Complete	N/A

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
32	PORV Setpoint	Perform analyses and modifications to reduce the likelihood of PORV actuations during anticipated transients per IE Bulletin 79-05B.	NUREG-0680 AMD 78 IR-81-21 IR-82-15 IR-83-09	Complete	Yes
33	Manual Reactor Trip	Provide procedures and training for prompt manual reactor trip for transients resulting in pressure increases per IE Bulletin 79-05B.	NUREG-0680	Complete	N/A
34	Anticipatory Reactor Trip	Provide for NRC approval a design review and schedule for implementation of a safety grade automatic anticipatory reactor scram for loss of feedwater, turbine trip or significant reduction in steam generator level. (IE Bulletin 79-05B).	NUREG-0680 AMD 78 IR-80-31 IR-81-03 IR-81-21 IR-82-14 IR-81-11 IR-81-28 IR-82-06 IR-82-26	Complete. Test during PET to be implemented by license condition.	No (Reg. 1) PET (9/83)
35	Prompt Reporting	Review prompt reporting procedures to assure NRC is notified within one hour of the time the reactor is not in a controlled condition per IE Bulletin 79-05B.	NUREG-0680	Complete	N/A
36	Tech Spec Changes	Propose changes as required to those Technical Specifications which must be modified as a result of implementing IE Bulletin 79-05B.	AMD 78 IR-82-19	Complete	No (Reg. 1) 6/83

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
37	RCP Trips	Upon reactor trip and initiation of HPI, immediately trip all RCPs. Provide two license operators to do this. (IE Bulletin 79-05C).	NUREG-0680	Complete	N/A
38	Small LOCA Analyses	Perform and submit a report of small LOCA analyses per IE Bulletin 79-05C.	NUREG-0680	Complete	N/A
39	Operator Action-RCP Trips	Based on the analyses of item 38, develop guidelines for operator action for LOCA and non-LOCA transients per IE Bulletin 79-05C.	NUREG-0680	Complete	N/A
40	Reactor Trip Training	Based on the guidelines developed for item 39, revise emergency procedures and train all operators. (IE Bulletin 79-05C).	NUREG-0680	Complete	N/A
41	Inadequate Core Cooling	Provide analyses and develop guidelines and procedures related to inadequate core cooling per IE Bulletin 79-05C.	NUREG-0680	Complete	N/A
42	<u>Emergency Preparedness</u>	Improve emergency preparedness per order item 3 of Commission Order dated 8/9/79.	NUREG-0746 IR-81-20	Complete	Yes
43	<u>Separation of IMI-1 and IMI-2</u> Liquid Radwaste	Isolate specific liquid radwaste connections between units.	NUREG-0680 IR-82-21 IR-83-01	Complete	Yes

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
44	Gaseous Radwaste Leak Test	Perform a helium leak test of the gaseous waste processing system.	NUREG-0680 IR-82-21 IR-83-11	Complete	Yes
45	Fuel Handling Building Isolation	Modify the TMI-1 fuel handling building layout to isolate the fuel handling floor from the auxiliary building and control building.	NUREG-0670 IR-82-21	Complete	Yes
46	Fuel Handling Building Ventilation Modification	Modify auxiliary building and fuel handling building ventilation system by adding two leak tight dampers.	NUREG-0680 IR-82-21	Complete	Yes
47	Hittman System	Install interim (Hittman) radwaste solidification system.	NUREG-0680 IR-81-34 IR-82-03 IR-82-21	Complete	Yes
48	Trash Segregation	Segregate trash during collection to insure TMI-1 and TMI-2 wastes remain separate.	NUREG-0680 IR-82-21	Complete	Yes
49*	(4) Monitoring System	Verify complete separation of TMI-1 and TMI-2 process and effluent radiological monitoring systems.	NUREG-0680	Complete	N/A
50	Independent TMI-2 Sampling System	Construct an independent radioactive liquid sampling system for TMI-2.	NUREG-0680 IR-82-21	Complete	Yes
<u>Waste Management</u>					
51	Liquid and Gaseous Radwaste Systems	Demonstrate that the waste management capability including storage and processing for liquid and gaseous wastes is adequate to assure safe operation of TMI-1 and that TMI-1 waste handling capability is not relied on by operations at TMI-2.	NUREG-0680	Complete	N/A

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
52	Solid Radwaste System	Demonstrate the waste management capability, including storage and processing for solid wastes is adequate to assure safe operation of IMI-1 and that IMI-1 waste handling capability is not relied on by operations at IMI-2 (Expand low activity solid waste storage).	NUREG-0680 IR-82-21 IR-83-01	Complete	Yes
<u>Managerial Capability</u>					
(3) 53*	Training of Operating Staff	Perform retraining of operating staff per numerous requirements summarized in NUREG-0680 page C6-5.	NUREG-0680	Complete	N/A
(3) 54*	Facility Procedures	Revise facility procedures to conform with requirements of CLI 80-5, IE Bulletins 79-05A, B, C and NUREG-0578.	NUREG-0680	Complete	N/A
(3) 55*	Operational Quality Assurance Program	Demonstrate the adequacy of the Operational Quality Assurance Program.	NUREG-0680	Complete	N/A
(3) 56*	Plant Maintenance	Demonstrate the adequacy of the plant maintenance support organization.	NUREG-0680 IR-80-21 IR-80-19	Complete	Yes
(3) 57*	Command and Administrative Structure Review	Determine whether the licensee's command and administrative structure at both the plant and corporate levels, is appropriately organized to assure safe operation of IMI-1.	NUREG-0680 IR-80-19	Complete	Yes

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
58* (3)	Operations and Technical Staff Qualifications Review	Determine whether the operations and technical staff of IMI-1 is qualified to operate Unit 1 safely (adequacy of the facility's maintenance program should be considered).	MUREG-0680 IR-80-21	Complete	Yes
59* (3)	Radwaste Operator Qualifications	Determine whether the IMI-1 Radiation Waste System is appropriately staffed with qualified individuals to ensure safe operation of the facility.	MUREG-0680 IR-80-21	Complete	Yes
60* (3)	Finance & Technical Department Relationship	Determine whether the relationship between licensee's corporate finance and technical departments is such as to prevent financial considerations from having an improper impact upon technical decisions.	MUREG-0680	Complete	N/A
61* (3)	Qualifications of Safety Review Groups	Determine whether the licensee has made adequate provision for groups of qualified individuals to provide safety review of and operational advice regarding IMI-1.	MUREG-0680	Complete	N/A
62* (3)	Comparison of IMI-1 Infractions with Industry-wide Statistics	Determine what, if any, conclusions regarding licensee's ability to operate IMI-1 safely can be drawn from a comparison of the number and type of past infractions of NRC regulations attributable to the Three Mile Island Units with industry-wide infraction statistics.	MUREG-0680	Complete	N/A

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
(3) 63*	Comparison of TMI-1 LERs with Industry-wide Statistics	Determine what, if any, conclusions regarding licensee's ability to operate TMI-1 safely can be drawn from a comparison of the number and type of past Licensee Event Reports ("LER") and the licensee's operating experience at the Three Mile Island Units with industry-wide statistics on LER's and operating experience.	NUREG-0680	Complete	N/A
(3) 64*	Review of Management Actions During the TMI-2 Accident	Determine whether the actions of Licensee's corporate or plant management (or any part or individual member thereof) in connection with the accident at TMI-2 reveal deficiencies in the corporate or plant management that must be corrected before TMI-1 can be operated safely.	NUREG-0680 IR-83-10	Open. (NRR, 01)	TBD
(3) 65*	Technical Capability Review	Determine whether licensee possesses sufficient in-house technical capability to ensure the simultaneous safe operation of TMI-1 and clean-up of TMI-2.	NUREG-0680	Complete	N/A
66	Management & Technical Capability	Assess management and technical capability and the training of the operations staff (Training Manual).	NUREG-0680 IR-80-19 IR-81-33 IR-82-16	Complete	N/A
67	Safety Review	Licensee make acceptable provisions for reviews and audits of changes to plant design and procedures (EP-013 and AP-1032). (See also item 106)	IR-80-19 IR-81-33 IR-82-09 IR-82-16 IR-82-18 IR-83-02	Complete	Yes

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
68	Health Physics Qualifications Review	Determine whether the TMI-1 Health Physics program is appropriately organized and staffed with qualified individuals to ensure safe operation of the facility.	NUREG-0680 IR-83-04	Complete	Yes
69	Training for Mitigating Core Damage	Complete training program prior to exceeding 5% power.	NRC ltr. dtd. 4/22/81 IR-81-33 IR-83-06	Complete	Yes
(2) 70*	Training Program Description	Prepare program description for training per item 69.	NRC ltr. dtd. 4/22/81	Complete	N/A
(2) 71*	Inspector Views	Resolve inspector concerns regarding quality of plant management, corporate management, staffing, organization and resources.	NUREG-0600 IR-80-19 IR-80-21 IR-80-22 NUREG-0680	--	No (Reg. 1) 7/83

Lessons Learned-Short Term

2.1.1 Emergency Power Supply

72	Pressurizer Heaters	Provide redundant emergency power for the minimum number of pressurizer heaters required to maintain natural circulation in the event of a loss of offsite power. (See also item 110).	NUREG-0680 IR-81-08 IR-81-28 IR-82-01 IR-82-19 IR-82-26	Complete	Yes
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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
73	Pressurizer Level, Relief and Block Valves	Provide emergency power to the control and motive power systems for the power-operated relief valve and associated block valve and to the pressurizer level indication instrument channels.	NUREG-0680	Complete	N/A
74	Relief Valve Testing Program Description	Commit to provide performance verification by full scale prototypical testing for all relief and safety valves.	NUREG-0680	Complete	N/A
75	Valve Position Indication	Provide in the control room either a reliable, direct position indication for the PORV and SRVs or a reliable flow indication devices downstream of the valves.*	NUREG-0680 IR-80-04 IR-80-05 IR-81-28 IR-81-22 IR-82-21 IR-82-26 IR-81-04 IR-81-24 IR-82-19	Complete	Yes
76	Inadequate Core Cooling	Perform analyses and implement procedures and training for prompt recognition of low reactor coolant level and inadequate core cooling using existing reactor instrumentation (flow, temperature, power, etc.) or short-term modifications of existing instruments. (See also item 113). (Note: License condition required by ALAB-729, p.176)	NUREG-0680 NUREG-0752 IR-81-10 IR-81-28 IR-82-14 IR-82-19 IR-82-26 IR-82-01 IR-82-10 IR-81-14 IR-81-24 IR-82-07 IR-82-17	Review Seismic qualification requirements. Estimated completion 6/83 (NRR). Test during PET to be implemented by license condition.	No (Reg. 1) PET (9/83)

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
77	Containment Isolation	Provide containment isolation on diverse signals, review isolation provisions for non-essential systems and revise as necessary, and modify containment isolation designs as necessary to eliminate the potential for inadvertent reopening upon reset of the isolation signal.	NUREG-0680 NRC itr. dtd. 4/22/81 IR-82-01 IR-82-07 IR-82-19 IR-82-26 IR-80-05 IR-80-31 IR-81-04 IR-82-07 IR-82-24 IR-81-05 IR-83-02	Complete.	No (Reg. 1) 6/83
78	Dedicated Hydrogen Penetrations	Provide dedicated penetrations and isolation systems that meet redundancy and signal failure requirements.	NUREG-0680	Complete	N/A
79	Integrity of Systems Outside Containment	Perform leakage rate tests on systems outside containment that process primary coolant and could contain high level radioactive materials. Develop and implement a periodic testing and preventative maintenance programs.	NUREG-0680	Complete. Test during HFT (8/83).	No (Reg. 1) 9/83
80	Plant Shielding	Perform a design review of the shielding of systems processing primary coolant outside of containment. Determine any areas or equipment that are vital for post-accident occupancy or operation and assure that access and performance will not be unduly impaired due to radiation from these systems.	NUREG-0680	Complete	N/A
81	EFW Auto-Initiation	Provide automatic initiation of all EFW systems. (See also items 1 and 121).	NUREG-0680 IR-80-26 IR-82-01 IR-81-28 IR-82-26	Complete. Test during PET to be implemented by license condition.	No (Reg. 1) PET (9/83)

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
82	EFW Flow Indication (Control Grade)	Provide indication in the control room of EFW flow for each steam generator. (See also item 5).	NUREG-0680 IR-02-17 IR-83-01 IR-83-02 IR-83-06	Licensee installing safety grade system per long-term requirement. Installed modification unsatisfactory, replacement modification required. Estimated review completion 8/83 (NRR).	No (Reg. 1) 8/83
83	Post-Accident Sampling	Review and upgrade the capability to obtain samples from the reactor cooling system and containment atmosphere under high radioactivity conditions. Provide the capability for chemical and spectrum analysis of high-level samples on site.	NUREG-0680	Complete	N/A
84	Radiation Monitors	Install safety-grade in containment radiation monitors.	NUREG-0680 IR-82-01 IR-83-04	Complete.	No (Reg. 1) 7/83
85	Gaseous Effluent Monitors (Noble Gas)	Install high-range gaseous effluent monitors. (See also item 114).	NUREG-0680 IR-82-01 IR-83-04	Review complete. Licensee estimates 6/83 modification completion.	No (Reg. 1) 7/83
86	Radiiodine/ Particulate Sampling	Install expanded range radiiodine/particulate sampling system. (See also item 115)	NUREG-0680 IR-82-01 IR-82-26 IR-83-04	Complete.	No (Reg. 1) 7/83
87	Iodine Instrumentation	Provide instrumentation for accurately determining in-plant airborne radiiodine concentrations to minimize the need for unnecessary use of respiratory protection equipment.	NUREG-0680 IR-83-04	Complete	Yes
88	Shift Supervisor	Review plant administrative and management procedures and revise as necessary to assure that reactor operations command and control responsibilities and authority are properly defined.	NUREG-0680	Complete	N/A

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
89	Shift Technical Advisor	Provide on shift a qualified person (the shift technical advisor) with a bachelor's degree or equivalent in a science or engineering discipline and with specific training in the plant response to off-normal events and in accident analysis of the plant.	NUREG-0680	Complete	N/A
(4) 90*	Shift turnover	Review, revise and evaluate effectiveness of plant procedures as necessary to assure that a shift turnover checklist is provided and required to be completed and signed by the on-coming and off-going individuals responsible for command of operations in the control room.	NUREG-0680	Complete	Yes
91	Control Room Access	Assure that access to the control room under normal and accident conditions is limited to those persons necessary to the safe command and control of operations.	NUREG-0680	Complete	N/A
92	Onsite Technical Support Center	A separate technical support center shall be provided for use by plant management, technical, and engineering support personnel.	NUREG-0680 NUREG-0746 IR-83-02	Complete	Yes
93	Onsite Operational Support Center	Establish and maintain an operational support center outside the central room.	NUREG-0680 NUREG-0746	Complete	Yes
<u>ASLB Management PID of 8/27/81</u>					
(1) 94*	Human Factors Components	Implement Human Factors items cited in GPU letter dated 1/21/81, (Hukill to Reid) that were to be done prior to restart and prior to 1983.		Estimated review completion 6/83 (NRR).	No (NRR) 6/83
95	Accuracy of Maintenance Records	Staff perform inspections and licensee perform actions described in Section II.D of 8/27/81 PID.	IR-81-27 IR-83-02	Complete.	Yes

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
(1) 96*	Overtime Policy	Verify continued implementation of policy set forth in CPU memo of February 1980 (PID paragraph 343).	IR-81-01	Estimated review completion 6/83 (NRR).	No (NRR) 6/83
(1) 97*	IOSRG	Document proper staffing of Independent Off-Site Review Group (IOSRG).		Estimated review completion 6/83 (NRR/Reg. 1).	N/A
98	Category T	Licensee administer Category T examinations to license candidates.	Board Notification BN-82-21 of 3/5/82	Complete	N/A
99	TMI-2 Accident Training	Licensee demonstrate that all licensee operators received three days additional training on TMI-2 accident.	IR-82-16	Complete	Yes
100	Simulator Examinations	Licensee demonstrate that all operators not previously NRC licensed have completed an NRC exam at B&W simulator.	BN-82-21 of 3/5/82	Complete	N/A
101	CRT Simulator	Licensee demonstrate that a CRT part-task simulator, which displays temperature and pressure, is available at TMI-1.	IR-81-33	Complete	Yes
102	Replica Simulator	Licensee prepare for bids and distribute specifications for a TMI-1 exact replica simulator. Make reasonable effort to install by 1985.	NRC ltr. dtd. 12/22/82	Complete	N/A
103	Contract for Basic Principles Trainer	Licensee demonstrate that a basic principles trainer for TMI-1 is contracted to be installed in 1982.	NRC ltr. dtd. 12/22/82	Complete	N/A

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
104	Emergency Director/ Support Director Training	Licensee demonstrate that members of senior management (since July 1, 1979) who are designated to act as Emergency Directors or Emergency Support Director, have received a formal training course.	IR-83-02	Complete	Yes
105	License Conditions for Shift Manning	At time of restart, staff impose and enforce license conditions per PID Section II.M.9(a)-(g) for operation at TMI-1 concerning shift manning.	--	NRR issue license condition (8/83).	No (Reg. 1) 8/83
106	Operating Information Management System	Licensee provide and maintain a management system to perform functions per PID Section II.M.10(a)-(c). (See also item 67).	IR-80-19 IR-81-33 IR-82-09 IR-82-16 IR-82-18 IR-83-02	Complete	Yes
<u>ASLB Plant Design, Procedures, and Modifications PID of 12/14/81</u>					
107	Natural and Forced Circulation	Install new shield walls between the HPI piping and motor control centers.	NUREG-0680 IR-82-13	Complete	Yes
108	Abnormal Transients Operating Guidelines	Staff review revised A74G program and report to Commission on reasonable progress.	NRC ltr. dtd. 10/26/82	Complete	N/A
109	Pressurizer Heaters	Demonstrate reactor coolant pressure control using HPI system.	IR-83-01	Complete	Yes
110	Connecting Pressurizer Heaters to Diesel - Procedures	Staff verify procedures for reconnecting pressurizer heaters. (See also item 72). (Note: License condition required per ALAB-729).	NUREG-0680	Complete	N/A
111	Connecting Pressurizer Heaters to Diesel - Demonstration	Licensee demonstrate connection of heaters to emergency buses; staff monitor test and evaluate results. (Note: Staff advise Commission if test disables emergency power supply. (ALAB-729)).	IR-83-01	Complete. Test during PET to be implemented by license condition.	No (Reg. 1) PET (9/83)

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
112	Computers - Operator Reliance	Staff verify that operators do not rely solely on computer information for making operational decisions	IR-82-18	Complete	Yes
113	Computers - Incore Thermocouple Backup	Install and have operational data logging or recording instrument displays of certain in-core temperature information (before 5% power -11c. condition). System must be safety-grade per ALAB-729, p. 176. (See also item 76).	NUREG-0680 NUREG-0752 IR-81-10 IR-81-28 IR-82-14 IR-82-19 IR-82-26 IR-82-01 IR-82-10 IR-81-14 IR-81-24 IR-82-07 IR-82-17	NRR estimates 8/83 review completion. NRR issue license condition before 5% power. Test during PEI.	No (Reg. I) PEI (9/83)
114	In-Plant Instrument Ranges - Effluent	Install extended range noble gas effluent monitors. (See also item 85).	NUREG-0680 IR-83-04	Review complete. Licensee estimates 6/83 modification completion.	No (Reg. I) 7/83
(2) 115*	In-Plant Instrument Range - Sampling	Install expanded sampling system capability for monitoring radioiodine and particulates. (See also item 86).	NUREG-0680 IR-82-26 IR-83-04	Complete.	No (Reg. I) 7/83
116	Safety-System Status Panel - Valve Lineup	Licensee perform complete review of safety system valve lineup. Staff perform independent verification. (See also item 23).	--	Perform verification at HFT (8/83)	No (Reg. I) 9/83
117	Safety-System Status Panel - Administrative Controls	Staff review licensee compliance with NUREG-0737 requirement to upgrade administrative controls for monitoring and verifying system status prior to restart.	NRC ltr. dtd. 4/22/81 IR-82-16 IR-83-02	Complete	Yes

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Number	Item	Definition	Completion Documentation	Review Status/Remarks	NRC Inspection Complete
118	Control Room Design Review	Licensee perform NUREG-0752 recommended corrections, staff review modifications against criteria of NUREG-0752 and Supplement 1. (See also item 125).	NUREG-0752 IR-82-17 IR-82-26 IR-81-24 IR-82-19 IR-83-09 IR-83-14	Complete	Yes
119	Additional LOCA Analysis	Install HPI system cavitating venturis and cross-connect.	IR-81-19 IR-82-07 IR-82-10 IR-82-26	Complete. Test during HFT. (8/83)	No (Reg. I) 9/83
120	Pressurizer Level Instrumentation	Upgrade pressurizer level instrumentation by providing redundant vital power supplies.	IR-82-26	Complete	Yes
121	EFW Reliability	Provide auto initiation of EFW. (See also items 1 and 81). (Note: System need not be safety grade at restart so long as an operator is dispatched to EFW station.) (ALAB-729).	NUREG-0680 IR-80-26 IR-82-01 IR-81-28 IR-82-26	Complete. Test during PET to be implemented by license condition.	No (Reg. I) PET (9/83)
122	Prevent EFW Isolation by Rupture Detection System Failure	Licensee propose means for preventing feedwater isolation due to failure in rupture detection system. (Reasonable progress). (See also item 153).	NRC ltr. dtd. 11/10/82 IR-83-01	Complete	Yes
123	Evaluate Organization and Management Improvements	Perform evaluation per Item I.B.1.2 of NUREG-0694.	NRC ltr. dtd. 4/22/81	Complete	N/A
124	NSSS Vendor Review of Procedures	Perform review per Item I.C.7 of NUREG-0694.	NRC ltr. dtd. 4/22/81 IR-82-16	Complete	Yes

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
125	Control Room Design Review	Perform review per Item I.D.1 of NUREG-0694. (See also item 118).	NUREG-0752 IR-82-17 IR-82-26 IR-81-24 IR-82-19 IR-83-09	Complete	Yes
126	Training During Low Power Testing	Perform training per Item I.G.1 of NUREG-0694.	NRC ltr. dtd. 4/22/81 NRC ltr. dtd. 12/21/82 IR-81-33	Complete. Test during PET to be implemented by license condition.	No (Reg. 1) PET (9/83)
127	Pilot Monitoring of Selected Emergency Procedures	Perform monitoring per Item I.C.8 of NUREG-0694.	NRC ltr. dtd. 4/22/81	Complete	N/A
128	Equipment Qualification	Staff certify to Commission a report on licensee's compliance with CL1-80-21 with respect to radiation.	1/28/82 staff filing to Commission	Complete	N/A
129	Equipment Qualification	Comply with six equipment qualification conditions of PID paragraph 1163.	NRC ltr. dtd. 10/8/82 IR-83-02	Complete	Yes
130	Flood Level Calculations	Staff perform an independent review of licensee's calculations of flood level in the reactor building. Staff conduct review of operational limitations that must be imposed.	NRC ltr dtd. 7/26/82 IR-81-28 IR-82-03 IR-82-07 IR-82-26 IR-82-06 IR-82-19	Staff evaluation estimated for 6/83 completion (NR).	Yes

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
131	Position on Cold Shutdown	Licensee environmentally qualify a single path to cold shutdown or bring the matter to the attention of the Commission.	1/28/82 staff filing to Commission	Complete	N/A
132	Ventilation Separation Program	Licensee submit to the staff a program designed to test the adequacy of its phase 1 ventilation separation program such that staff can certify that a satisfactory test program has been implemented.	NRC ltr. dtd. 3/9/83	Complete	N/A
<u>ASLD Emergency Planning PID of 12/14/81</u>					
(1) 133*	Emergency Planning Health Physics Matters	A. Complete 1 training cycle of the Emergency Plan training program. B. Verify upgrade of Reactor Building Alarm. C. Include high range noble gas monitor readings and an radiiodine effluent analyses in the dose assessment scheme. D. Implement procedures for collecting and analyzing absorbent media for radioiodine and gaseous effluents under accident conditions (Paragraphs 1404, 1479).		Review scheduled for 6/83 completion (IE).	No (IE) 6/83
(1) 134*	EALs	Verify and document that the General Emergency and reactor coolant system activity Emergency Action Levels (EALs) have been revised to be consistent with NUREG-0654.		Review scheduled for 6/83 completion (IE).	N/A
(1) 135*	Alert EAL	Verify and document that the reactor coolant system EAL for an Alert precisely conforms to the 300 MCl/ml equivalent I-131 criteria in NUREG-0654.		Review scheduled for 6/83 completion (IE).	N/A
(1) 136*	Dose Projection	Verify and document revision of procedures for off-site dose projection to include a curve of estimated containment leak rate as a function of containment design pressure and actual measured pressure.		Review scheduled for 6/83 completion (IE).	N/A

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
137*	(1) Emergency Response Training	<p>A. Verify and document completion of one full iteration of Emergency Organization Training for offsite support organizations including local fire depts, ambulance services, local police depts. State Police and Middletown police get specific training.</p> <p>B. Verify and document Radiation Management Corp. Annual Training of Hospital and ambulance personnel.</p> <p>C. Verify and document formal training course on plant design features for licensee senior management who joined the licensee since 6/9/79 and who will serve as Emergency Director or Emergency support Director.</p>		Review scheduled for 6/83 completion (1E).	No (1E) 6/83
138	EOF Staffing Requirements	Implement EOF Support Director staffing requirements of PID 2010a and 1381 for restart.	--	Commission reviewing ASLAB decision (CL1-83-7).	--
139	County and PEMA EP Brochure Review	The staff shall review any changes made in their final risk county brochures and the PEMA Pamphlet on emergency preparedness, and advise the Commission prior to restart of the impact of the revisions on the intended purpose of these documents.		Review scheduled for 6/83 completion (1E).	N/A
140	EP Brochure	Updated PEMA and 5 risk county brochures distributed (Public Information on Emergency Preparedness).		Review scheduled for 6/83 completion (1E).	N/A
141	EP Brochures to Transient Locations	Updated county brochures distributed to transient (hotels, motels, parks, employers) locations.		Review scheduled for 6/83 completion (1E).	N/A

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
142	Major Employer Briefings	At least be underway for licensee briefings to major employers and operators of transient locations.		Review scheduled for 6/83 completion (IE).	N/A
143	Siren System Test	Siren System audibility test. Staff certify results and need for supplemental alerting.		Review scheduled for 6/83 completion (IE).	N/A
144	Communications Drill	Conduct communications drill to stress system.		Review scheduled for 6/83 completion (IE).	N/A
145	School District Plans	Staff certify when plans for EP-Z school districts are completed and reviewed for adequacy.		Review scheduled for 6/83 completion (IE).	N/A
<u>ASLB Reopened Proceeding on Cheating PIB of 7/14/82</u>					
(5) 146*	Operator Certification	Verify and document implementation of a procedure for certification of operators by signed statements from Training Personnel that operator candidates (to obtain or renew a license) have completed training requirements.		Review scheduled for 6/83 completion (NRR).	N/A
(5) 147*	Qualifications of Instructors	Licensee establish criteria for qualifications of instructors.	--	Review scheduled for 6/83 completion (NRR).	N/A
(5) 148*	Supervision of Start-up and Test Director	Any participation by Start-up and Test Director in start-up and test at TMI-1 is to be directly supervised by a qualified GPU official.	IR-82-19	Complete	Yes

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<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
	<u>ASEAB Imposed Emergency Planning Requirements</u>				
(5) 149*	Agricultural Brochure	Commonwealth agriculture brochure shall be distributed to all farmers in 10 mile plume exposure EPZ.		Review scheduled for 6/83 completion (IE).	N/A
	<u>ASLB Management PID of 8/27/81 (Additional Items)</u>				
(2) 150*	Feedback of Operating Experience	NUREG-0737, I.C.5 "Procedure for Feedback of Operating Experience to Plant Staff" must be enforced at TMI-1 if restart is permitted.		Estimated review completion 6/83. (NRR/Reg. 1)	TBD
(4) 151*	Start-up Test Program	Staff certify to Commission whether licensee has Startup Test Program which will provide additional operator training to satisfy the issue of whether to treat TMI-1 differently from other operating reactors because of the prolonged shutdown.		Estimated review completion 6/83. (NRR)	N/A
	<u>ASLB Emergency Planning PID of 12/14/81 ADDITIONAL ITEMS</u>				
(4) 152*	Dauphin County Plan	Staff certify to Commission that Dauphin County plan has been revised so that there is no suggestion in the plan that school students will be returned to their homes by bus in the early stages of an accident.		Review scheduled for 6/83 completion (IE).	N/A
(5) 153*	Main Steam Rupture Detection System Long-Term Solution	Licensee submit proposed long-term solution to Commission prior to restart. (See also item 122).		Licensee submittal TBD. Commission review TBD.	--

<u>Number</u>	<u>Item</u>	<u>Definition</u>	<u>Completion Documentation</u>	<u>Review Status/Remarks</u>	<u>NRC Inspection Complete</u>
(5) 154*	Subcooling Margin Instrumentation Error	Ensure subcooling margin instrument error does not exceed 20°F.		Licensee submittal TBD. NRR review completion TBD.	No (Reg. 1) TBD

*Additional item, not previously listed in SECY 82-250.

() Refers to basis for item being added to "certification" list.

- (1) Cited in hearing record as key assumption used by Board for its decision.
- (2) Subdividing item for clarity/completeness.
- (3) Not previously included because these items were already completed when SECY 82-250 was issued.
- (4) Items that had been previously omitted.
- (5) ASLB/ASLAB - Imposed after issuance of SECY 82-250.

TABLE 2

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NON-CERTIFICATION ITEMSPLANT SPECIFIC ACTIONS

ITEM	LICENSEE ACTION	NRC ACTION
1. Steam Generator Repair	C	(1) (3)
2. Reactor Building Purge TS (Test Flow Rates)	C	(1) (3)
3. H ₂ Recombiner TS	C	(1) (3)
4. Appendix J (H ₂ Recombiner Penetration Surveillance)	C	(2) (3)
5. Aux./Fuel Handling Building Ventilation Flow Rates	C	(1) (3)
6. Reactor Vessel Surveillance Capsule TS	C	(2) (3)
7. Snubber Detection (Correct TS to Reflect Plant Status)	C	(2) (3)
8. Post-Accident Monitoring Instrumentation TS Revisions	C	(2) (3)
9. Administrative Changes to TS	C	(2) (3)
10. Safety Injection System By-Pass Setpoint TS Change	C	(1) (3)
11. Cycle 5 Reload Analysis (Pending Licensee Submittal)	Est. 6/83 submittal	(1) (3)
12. IST Review	C	(2)
13. Additional Water Storage Capacity	C	(2)
14. Main Condenser Off-Gas Iodine Sampler	N/A	(2)
15. Requalification Program Changes	C	(1)

Codes

- (1) Scheduled for issuance prior to restart.
(2) Under staff review on a schedule consistent with that for other operating reactors.
(3) Requires license amendment.
C Complete

NON-CERTIFICATION ITEMS (CONT'D)TMI-1 NUREG-0737

<u>ITEM</u>	<u>LICENSEE ACTION</u>	<u>NRC ACTION</u>
*1. I.C.1 Procedures for Transients and Accidents	C	(1)
*2. I.D.1 Control Room Design Review	C	(2)
*3. I.D.2 Plant Safety Parameters Display Console	C	(2)
*4. III.A.1.2 Upgrade Emergency Support Facilities	C	(2)
*5. III.A.2.2 Meteorological Data	C	(2)
6. II.B.1 RCS Vents	C	(1)
7. II.B.3 Post Accident Sampling	C	(2)
8. II.E.1.1 AFW Evaluation	C	(1)
9. II.E.1.2 AFW Initiation & Flow Indication	C	(2)
10. II.F.1.4 Containment Pressure Monitor	C	(2)
11. II.F.1.5 Containment Water Level Monitor	C	(2)
12. II.F.1.6 Containment H ₂ Monitor	C	(2)
13. II.F.2 Inadequate Core Cooling (Per Order dated 12/10/82)	C	(2)
14. II.K.2.17 RCS Voiding	C	(2)
15. II.K.3.1 Auto PORV Isolation	TBD (subject to results of item 16 review)	TBD (1)

*NUREG-0737, Supplement 1 item, licensees proposed schedule under staff review.

TMI-1 NUREG-0737-ACTIONS (CONT'D)

ITEM	LICENSEE ACTION	NRC ACTION
16. II.K.3.2 Report on PORV Failures	C	(2)
17. II.K.3.30 SB LOCA Methods	C	(1) subject to integral test facility result
18. II.K.3.31 SB LOCA Plant Unique Analysis	TBD (subject to staff approval of item 17).	TBD (1)
19. III.D.3.4 Control Room Habitability	C	(2)

Codes

- (1) Pre-implementation review on a schedule consistent with that for other operating reactors.
- (2) Post-implementation review on a schedule consistent with that for other operating reactors.

C Complete

NON-CERTIFICATION ITEMS (CONT'D)MULTI-PLANT ACTIONS (MPA)

MPA ITEM	LICENSEE ACTION	NRC ACTION
1. B-17 TS Hyd. Snubber Surveillance	C	(1) (2)
2. B-24 Purge & Vent Valves	C	(1) (2)
3. B-41 Fire Protection Appendix R	C	(1)
4. B-48 Adequacy of Voltage Dist.	Est. 3/83 submittal	(3)
5. B-57 DHR Capabilities TS	C	(1) (2)
6. B-59 Masonry Walls	C	(1)
7. B-66 Natural Circulation (RV Head = Voiding)	C	(1)
8. B-72 NUREG-0737 TS	C	(1) (2)
9. B-73 Pressurized Thermal Shock	C	(1)
10. C-10 Control of Heavy Loads	Est. 12/83 submittal (Phase I)	(3)
11. C-14 Seismic Qualification of AFW	C	(1)
12. D-17 Definition of Operable TS	Est. 9/83 submittal	(2) (3)
13. D-18 NUREG-0630 Cladding Model	Est. 6/83 submittal	(2) (3)
14. D-60 Equipment Qualification	C	(1)
15. G-1 Reactor Coolant Pump Trip	C	(1)

Codes

- (1) Under staff review on a schedule consistent with that for other operating reactors
 (2) Requires license amendment
 (3) Awaiting licensee submittal
 C Complete

TABLE 2 **DRAFT**

NON-CERTIFICATION ITEMS (CONT'D)

REGION I PRE-RESTART INSPECTION ITEMS

ITEM	LICENSEE ACTION	NRC ACTION
1. Fire Protection	Complete Modifications 7/ /83 (NRR denied scheduler exemption)	(1)
2. Loss of Non-Nuclear Instrumentation/ Integrated Control System Power	C	(1)
3. Loads off Safeguard Busses	Modification Complete. Resolve Outstanding Item (OI) 6/83	(1)
4. Technical Support Center Modifications	6/83	(1)
5. Waste Gas Disposal System Pipe Cracks	C	(1)
6. PORV Corrosion	C	(1)
7. Decay Heat/Core Flood Check Valve Leakage	Resolve OI 6/83	(1)
8. Reactor Building Spray Valve Modifications	C	(1)
9. Inservice Test (IST) of Restart Modification Valves	6/83	(1)
10. Auxiliary/Fuel Handling Building Ventilation System	6/83	(1)
11. Reactor Protection System Breakers	C	(1)
12. Reactor Coolant System (RCS) Loop Vents	6/83	(1)
13. Secondary System Power Modifications and Decontamination Facility	6/83	(1)
14. Emergency Feedwater (EFW) Flow Verification	Preop test after OTSG repair 7/83	(1)

REGION I PRE-RESTART INSPECTION ITEMS

<u>ITEM</u>	<u>LICENSEE ACTION</u>	<u>NRC ACTION</u>
15. Decay Heat Pump Bearing Failure	TBD	(7/83) (1)

Codes

(1) Complete inspection before restart.

C Complete