	CONTROL BLOCK:	L REQUIRED INFORMATION
7 1 1 8	9 R A N O 1 1 2 O O O O O O O O O	
7 1 1 8	REPORT L 6 0 5 0 0 0 3 1 3 7 0 5 2 0 8 SOURCE 60 61 00 KET NUMBER 68 69 EVENT DATE	3 18 10 15 11 10 10 12 12
10121	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES TO	
10131	10n 5/20/83, while raising steam generator level for wet layup, emergency feedwa	iter (EFW) pump P-7B indicated
10141	I that the flow appeared degraded. The facility was in cold shutdown at the time	of the occurrence. This
10151	loccurrence is reportable per Technical Specification 6.12.3.2.c. No similar oc	currences have been reported.
10161		
10171		
10181		
17 RE	SYSTEM CAUSE CAUSE CODE SUBCODE COMPONENT CODE	COMP VALVE SUBCODE SUBCODE B 15
33	EN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUE 12 12 12 10 10 10 10 12 17 12 18 18 18 19 12 12 12 10 10 10 10 10 12 17 12 18 18 18 18 18 18 18 18 18 18 18 18 18	PRIME COMP. COMPONENT 3 SUPPLIER MANUFACTURER 4 A 25 1 0 7 5 26 47
11101	The cause of the flow degradation was cavitation at high flows from suction foul	ling. A glove was found lodged
	In the suction of the pump. Inadequate administrative controls and personnel er	ror were apparently responsible
	for the failure. A spool piece was removed from the suction piping to allow ins	
	The glove was discovered and removed, and the spool piece was reinstalled. Both	
1 5 1 5 1 S	10 10 12 13 13 14 45 46 Operator Obs	ERY DESCRIPTION
1 6 1 1		TION OF RELEASE
	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION O O *O 37 Z 138 NA SONNEL INJURIES 11 12 13 SONNEL INJURIES	136 80 139
	NUMBER DESCRIPTION	
8 9 L	OSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION Z 142 NA S S S S S S S S S S S S S S S S S S	141
21 <u>0</u> 1. 1 8 9	PUBLICITY SSUED DESCRIPTION N 144 NA 10	NRC USE ONLY
N/	AME OF PREPARER Patrick Rogers	PHONE: (501) 964-3100
		TE22

NRC FORM 366 (7-77)

LICENSEE EVENT REPORT

U.S. NUCLEAR REGULATORY COMMISSION

EXHIBIT A

LER No. 50-313/83-013/03L-0

Occurrence Date: 5/20/83

Cause Description and Corrective Actions (Continued):

Actions to prevent recurrence will be determined based on the results of the investigation.



ARKANSAS POWER & LIGHT COMPANY POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000 June 10, 1983

1CANØ683Ø9

Mr. W. C. Seidle, Chief Reactor Project Branch #2 U. S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, Texas 76011

Subject: Arkansas Nuclear One - Unit 1

Docket No. 50-313 License No. DPR-51 Licensee Event Report No. 83-013/03L-0

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.2.c, attached is the subject report concerning flow degradation of emergency feedwater pump P-7B.

Very truly yours,

John R. Marshall Manager, Licensing

JRM: RJS: s1

Attachment

cc: Mr. Richard C. DeYoung Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission Washington, D. C. 20555

> Mr. Norman M. Haller, Director Office of Management & Program Analysis U. S. Nuclear Regulatory Commission Washington, D. C. 20555