

December 23, 1982

Mr. H. R. Denton, Director Office of Nuclear Reactor Regulation U. S. NUCLEAR REGULATORY COMMISSION Washington, D. C. 20555

Attention: Mr. R. A. Clark, Chief Operating Reactors, Branch 3

Gentlemen:

## DOCKET NOS. 50-266 AND 50-301 FURTHER RESPONSE TO NUREG-0737, ITEM II.D.1 RELIEF AND SAFETY VALVE TESTING POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

This is to provide the current status of our continuing evaluations and analyses regarding relief and safety valve operability in response to NUREG-0737 requirements. The enclosure to this letter provides the results of piping and support analyses for the existing installations at Point Beach Nuclear Plant, Units 1 and 2, and discussion of valve operability based upon EPRI Test Program results.

The detai piping and support analyses described in the enclosure have ic stified modifications which will be implemented to provide the stability of piping and supports to accommodate the calculate sale cruation loads during transient conditions. Conceptual sup ort designs and piping stress analyses using the conceptual designs have been completed for Unit 1 and are in progress for Unit 2. A detailed walkdown was performed during the recent Unit 1 refueling outage to verify that supports and support modifications can be installed as indicated by the conceptual study. Final support designs will be developed for Unit 1 and are expected to be installed during the steam generator replacement outage scheduled to begin in October 1983. A similar walkdown will be conducted in Unit 2 during the refueling and steam generator sleeving outage scheduled to begin in late March 1983. Based upon the walkdown, final support designs will be developed. We expect to complete the installation or modification of Unit 2 supports during the 1983 refueling and sleeving outage.

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Should you have questions concerning this submittal, please contact us.

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Very truly yours,

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Assistant Vice President

C. W. Fay

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Enclosure

Copy to NRC Resident Inspector

