

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT  
/0/1/ REPORT SOURCE /L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/5/0/3/8/3/ (8) 052683/ (9)

DOCKET NUMBER EVENT DATE REPORT DATE  
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On May 3, 1983, with Unit 2 in Mode C, suction to the "B" Residual Heat Removal /  
/0/3/ / (RHR) Pump was lost while transferring water from the Reactor Coolant System /  
/0/4/ / (RCS) to the Refueling Water Storage Tank (RWST) via the Refueling Purification /  
/0/5/ / (RP) System. The "A" Pump was secured and the "B" RHR Pump started but suction /  
/0/6/ / was not available. Since suction to the RHR Pumps was quickly restored and any /  
/0/7/ / RCS temperature increase was negligible the public health and safety were not /  
/0/8/ / affected. This event is reportable pursuant to T.S. 6.9.1.9.b. /

SYSTEM CAUSE CAUSE COMP. VALVE  
CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /C/F/ (11) /A/ (12) /A/ (13) /P/U/M/P/X/X/ (14) /B/ (15) /Z/ (16)  
LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION  
NUMBER REPORT NO. TYPE NO.

(17) /8/3/ /-/ /0/3/8/ / / /0/3/ /L/ /-/ /0/  
ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT  
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/X/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /Y/ (23) /Y/ (24) /N/ (25) /I/0/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Suction to the RHR pump was lost because the RCS was pumped below the established/  
/1/1/ / operating limit. Pumping of the RCS continued without adequate monitoring of the/  
/1/2/ / RCS level. The RCS was refilled and RHR pump suction restored. The responsible /  
/1/3/ / senior operator was reinstructed. /  
/1/4/ / /

FACILITY STATUS %POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)  
/1/5/ /H/ (28) /0/0/0/ (29) / NA / (30) /A/ (31) / OPERATOR OBSERVATION /

ACTIVITY CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)  
/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)  
/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES NUMBER DESCRIPTION (41)  
/1/8/ /0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION  
/1/9/ /Z/ (42) / NA /

PUBLICITY ISSUED DESCRIPTION (45) PDR ADOCK 05000339 S PDR NRC USE ONLY  
/2/0/ /N/ (44) / NA /

NAME OF PREPARER E. Wayne Harrell PHONE (703) 894-5151

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# Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

May 26, 1983

USNRC REGION II  
ATLANTA, GEORGIA  
83 MAY 31 AM 11:20

Mr. James P. O'Reilly, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, Georgia 30303

Serial No. N-83-060

NO/WFS: 11

Docket No. 50-339

License No. NPF-7

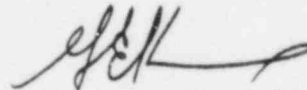
Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.	Applicable Technical Specifications
LER 83-038/03L-0	T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,



*fw* E. Wayne Harrell  
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)  
016 Phillips Bldg.  
U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

OFFICIAL COPY

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Description of Event

On May 3, 1983, with Unit 2 in Mode 6, the suction to the "B" RHR Pump was lost while pumping water from the reactor vessel to the Refueling Water Storage Tank (RWST) via the Refueling Purification (RP) System. The "A" RHR Pump was secured and the "B" RHR Pump started but adequate suction for the "B" Pump was not available. With both RHR Pumps unavailable the requirements of T.S. 3.9.8.1 were not met. This event is reportable pursuant to T.S. 6.9.1.9.b.

Probable Consequences of Occurrence

At the time of the event the Unit 2 was in Mode 6 and any RCS temperature increase was negligible. Therefore, the public health and safety were not affected.

Cause of Event

This event occurred because the Operator failed to properly monitor RCS level during pumpdown. The RP System was being utilized to pump down the Reactor Coolant System (RCS) to approximately 10 inches above centerline of the nozzles. Suction to the RHR pumps was lost when the RCS water level dropped below the established operating limits. Consequently, adequate NPSH was lost to the RHR pumps and resulted in cavitation in either RHR pump when running.

Immediate Corrective Action

Upon receiving the RHR low flow alarm and noticing RHR pump amps fluctuating the operating RHR pump and the RP pump were secured. The opposite RHR pump was started but was secured because of cavitation. Water was sluiced from the Refueling Water Storage Tank to the RCS, a RHR pump was restarted and proper RHR flow verified.

Scheduled Corrective Action

No further Corrective Action is scheduled.

Action Taken To Prevent Recurrence

Appropriate disciplinary action was taken.

Generic Implications

There are no generic implications to this event.