NRC FORM 366
(12-81)
10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011

CONTROL BLOCK: ☐☐☐☐☐☐ ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	A L B R F 3	②	0 0 - 0 0 0 0 0 - 0 0	③	4 1 1 1 1	④		⑤
	LICENSEE CODE		LICENSE NUMBER		LICENSE TYPE		CAT	

CON'T

0 1	REPORT SOURCE	L	⑥	0 5 0 0 0 2 9 6	⑦	0 4 1 8 8 3	⑧	0 5 1 7 8 3	⑨
				DOCKET NUMBER		EVENT DATE		REPORT DATE	

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 During normal reactor operation, both offgas post-treatment radiation monitors
0 3 (3-RM-90-265/266) became inoperable due to sample pump failure, contrary to the
0 4 requirements of T.S. 3.2.D.1.b&c. There was no effect on public health and
0 5 safety since the stack radiation monitor was operable and no
0 6 release limits were exceeded. A review of SI 4.8.B.1.a.1, Airborne Effluent
0 7 Release Rate, indicated no significant increase in building effluent activity
0 8 levels during this event.

	SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE		COMP. SUBCODE		VALVE SUBCODE	
0 9	I E	⑪	E	⑫	F	⑬	P U M P X X	⑭	X	⑮	Z	⑯

⑰ LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 2 7	/	0 3	L	—	0

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS	㉒	ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	㉖
C	⑱	Z	⑲	Z	⑳	Z	㉑	0 0 0 0		Y	㉓	N	㉔	L	㉕	M 2 7 0	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 The cause of this event was sample pump failure, probably due to normal end-of-
1 1 life operation. The redundant pump was placed in service within 25 minutes.
1 2 The Metal Bellows Corp., Model No. MB21, Part No. 28649, sample pump was
1 3 replaced on the next day. This is considered to be a random failure and no
1 4 recurrence control is required.

	FACILITY STATUS		% POWER		OTHER STATUS	㉚	METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	㉜
1 5	E	㉘	0 9 9	㉙	NA		A	㉛	Control Room Alarm	

	ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY	㉟	LOCATION OF RELEASE	㊱
1 6	Z	㉝	Z	㉞	NA		NA	

PERSONNEL EXPOSURES

	NUMBER		TYPE		DESCRIPTION	㊴
1 7	0 0 0	㊲	Z	㊳	NA	

PERSONNEL INJURIES

	NUMBER		DESCRIPTION	㊶
1 8	0 0 0	㊵	NA	

LOSS OF OR DAMAGE TO FACILITY

	TYPE		DESCRIPTION	㊸
1 9	Z	㊷	NA	

8305310054 830517
PDR ADOCK 05000296
S PDR

PUBLICITY

	ISSUED		DESCRIPTION	㊺	NRC USE ONLY
2 0	N	㊹	NA		☐☐☐☐☐☐☐☐☐☐☐☐

NAME OF PREPARER J. W. Burton, III

PHONE (205)729-0621

Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 83027 Technical Specification Involved 3.2.D.1

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 5/18/83

Event Narrative:

Unit 1 was in a refueling outage, unit 2 was operating at 94-percent power, and unit 3 was operating at 99-percent power. Only unit 3 was affected by the event. At approximately 1440 hours on April 18, 1983, the offgas post-treatment radiation monitor sample pump failed, causing both offgas post-treatment radiation monitors (3-RM-90-265/266) to become inoperable. The failure initiated a control room alarm. Plant operations personnel were immediately dispatched and placed the redundant post-treatment sample pump in service within 25 minutes. Technical Specification (T.S) 3.2.D.1 requires that both offgas post-treatment radiation monitors be operable during reactor operation. This requirement was not met for this 25-minute period. Because of this quick recovery, a shutdown (T.S. 3.2.D.1.c) was not required.

During the time both offgas post-treatment monitors were inoperable, the offgas pre-treatment and stack radiation monitors were operable. No significant increase in activity levels was detected by these monitors during this time. Additionally, a review of SI 4.8.B.1.a.1 (Airborne Effluent Release Rate) revealed no significant increase in building effluent activity during this time. This event had no adverse effect on the health or safety of the public.

The cause of this event was sample pump failure. The sample pump failure was apparently due to normal wear-out. The pump is a sealed unit and further investigation is not practical. The failed sample pump was replaced on April 19, 1983. This event is considered to be a random failure and no recurrence control is required.

* Previous Similar Events:

BFRO-50-259/78032

Retention: Period - Lifetime: Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

83 MAY 24 A10:53

REGION
A, GEORGIA

May 17, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFRO-296/83027

The enclosed report provides details concerning offgas post-treatment radiation monitors that became inoperable because of sample pump failure. This report is submitted in accordance with Browns Ferry unit 3 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Hubert Abercrombie

H. J. Green
Director of Nuclear Power

Enclosure
cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

OFFICIAL COPY

IE 22

/11