Tech. Spec. 6.9.A.2

Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier
Vice President Nuclear Operations
and Station Director

July 14, 1994
BECo Ltr. #94-078

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

JUNE 1994 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

L.J. Olivier

WJM/lam/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

JE24

OPERATING DATA REPORT

DOCKET NO. 50-293
DATE July 14, 1994
COMPLETED BY: W. Munro
TELEPHONE (508) 830-8474

OPERATING STATUS		NOTES
1.	Unit Name	Pilgrim I
2.	Reporting Period	May 1994
3.	Licensed Thermal Power (MWt)	1998
4.	Nameplate Rating (Gross MWe)	678
5.	Design Electrical Rating (Net MWe)	655
6.	Maximum Dependable Capacity (Gross MWe)	696
7.	Maximum Dependable Capacity (Net MWe)	670

8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	720.0	4343.0	118975.0
12.	Hours Reactor Critical	720.0	4050.0	116991.5
13.	Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14.	Hours Generator On-Line	720.0	3930.8	112719.8
15.	Hours Unit Reserve Shutdown	0.0	0.0	0.0
16.	Gross Thermal Energy Generated MWH)	1419936.0	7453368.0	198606312.0
17.	Gross Electrical Energy Generated (MWH)	487010.0	2556260.0	67202194.0
18.	Net Electrical Energy Generated (MWH)	468820.0	2460133.0	64593827.0
19.	Unit Service Factor	100.0	90.5	59.6
20.	Unit Availability Factor	100.0	90.5	59.6
21.	Unit Capacity Factor (Using MDC Net)	97.2	84.5	51.0
22.	Unit Capacity Factor (Using DER Net)	99.4	86.5	52.2
23.	Unit Forced Outage Rate	0.0	4.6	11.6

24. Shutdowns scheduled over next 6 months (type, date, and duration of each) - MCO-10 October (30 Days)
25. If shutdown at end of report period, estimated date of startup - UNIT OPERATING

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
DATE: July 14, 1994
COMPLETED BY: W. Munro
TELEPHONE: (508) 830-8474

MONTH June 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	661	17	659
2	660	18	661
3	660	19	525
4	658	20	660
5	659	21	660
6	660	22	661
7	662	23	660
8	660	24	658
9	660	25	659
10	558	26	660
11	659	27	630
12	660	28	662
13	660	29	661
14	662	30	661
15	661		
16	659		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR JUNE 1994

The unit started the reporting period at approximately 100 percent core thermal power (CTP). This power level was essentially maintained until June 10, 1994. At 1054 hours on June 10, 1994, the "A" Recirculation Pump Motor Generator (MG) Set tripped, and power was reduced to approximately 41 percent CTP at 1141 hours. At 1432 hours, following troubleshooting, the "A" Recirculation Pump MG Set was restarted and reactor power was increased. AT 1914 hours, 100 percent power was attained, and was essentially maintained until June 19, 1994, when power was reduced to approximately 47 percent CTP to facilitate a planned thermal backwash of the main condenser. Following completion of the backwash, power was increased to 100 percent CTP at approximately 1530 hours where it was essentially maintained until June 27, 1994. At 0400 hours on June 27, 1994, power was reduced to approximately 73 percent CTP due to a malfunction of the "A" 3rd Point Feedwater Heater Level Controller. Following maintenance to restore the feedwater heater to service, power was increased, reaching 100 percent CTP at approximately 0800 hours, where it was maintained for the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES
MONTH OF JUNE 1994

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.

2. Scheduled date for next refueling shutdown: April 1, 1995.

3. Scheduled date for restart following next refueling: May 26, 1995.

4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.

5. See #6.

6. The new fuel loaded during the 1993 refueling outage was of the same design as loaded in the previous refueling outage and consisted of 140 assemblies.

7. (a) There are 580 fuel assemblies in the core.

   (b) There are 1629 fuel assemblies in the spent fuel pool.

8. (a) The station is presently licensed to store 3859 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.

   (b) The planned spent fuel storage capacity is 3859 fuel assemblies.

9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 691 fuel assemblies.

MONTH JUNE 1994

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOC LER
Salt Service Water (SSW) System	SSW Pump P-208A Motor	Degraded performance	Age related degradation	Replaced P-208A motor with new motor of improved design. (Mfr. Reliance Electric) Balanced motor and pump per Procedure 3.M.1-15	None	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293

DOCKET NO: 50-293
NAME: Pilgrim I
DATE: July 14, 1994
COMPLETED BY: W. Munro
TELEPHONE: (508) 830-8474
REPORT MONTH: June 1994

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
7	6/19/94	S	0.0	B	N/A	N/A	N/A	N/A	Power reduction to facilitate a thermal backwash of the main condenser

1 2 3 4&5

F-Forced
S-Sched

A-Equip Failure
B-Main or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Operator Error
H-Other

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

Exhibit F & H
Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)