NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
01	LICENSEE CODE (2)	WIPBH2
01	LICENSE NUMBER (3)	00-00000-00
01	LICENSE TYPE (4)	41111
01	CAT (5)	
01 CON'T	REPORT SOURCE (6)	L
01 CON'T	DOCKET NUMBER (7)	05000301
01 CON'T	EVENT DATE (8)	042883
01 CON'T	REPORT DATE (9)	051183

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 04/18/83, after insulation had been removed from the "A" loop RTD
03 bypass line on Unit 2 for ISI purposes, Maintenance personnel found
04 evidence of a body-to-bonnet leak on RC-559A, the "A" loop RTD bypass
05 isolation valve. Further evaluation on 04/28/83 disclosed boric acid
06 wastage of 4 of 12 studs. This occurrence is reportable in accordance
07 with TS 15.6.9.2.A.3 as an abnormal degradation of the reactor coolant
08 pressure boundary.

Line	SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
09	CB	E	B	VALVEX	E	D

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	83	—	005	/	01	T	—	0

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
A	A	Z	Z	0000	Y	Y	N	V085

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 It is believed that a body-to-bonnet leak on valve RC-559A caused the
11 wastage of the 4 studs. All of the studs and nuts on RC-559A were
12 replaced and the bolting on the identical valve of the other loop was
13 examined yielding no recordable indications.
14

Line	FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
15	H	000	NA	B	Inservice Inspection

Line	ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z	Z	NA	NA

PERSONNEL EXPOSURES

Line	NUMBER (37)	TYPE (38)	DESCRIPTION (39)
17	000	Z	NA

PERSONNEL INJURIES

Line	NUMBER (40)	DESCRIPTION (41)
18	000	NA

LOSS OF OR DAMAGE TO FACILITY (43)

Line	TYPE (42)	DESCRIPTION
19	Z	NA

8305240350 830511
PDR ADOCK 05000301
S PDR

PUBLICITY

Line	ISSUED (44)	DESCRIPTION (45)
20	N	NA

NRC USE ONLY

NAME OF PREPARER C. W. Fay PHONE: 414/277-2811

ATTACHMENT TO LICENSEE EVENT REPORT NO. 83-005/01T-0

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 2
Docket No. 50-301

In preparation for inservice inspection of a weld on the "A" loop common RTD bypass return line, insulation was removed from the line revealing unusually high levels of surface contamination on the piping surface. An investigation by Maintenance personnel on April 18, 1983 as to the possible cause of the contamination revealed evidence of a body-to-bonnet leak on RC-559A, "A" loop isolation valve for the RTD bypass return line.

Valve RC-559A was disassembled by Maintenance and the studs examined on April 18. There was evidence of boric acid wastage on four of the twelve studs. The studs and nuts were replaced with new ones and the valve was reassembled on April 18. The following manufacturing data applies to the affected valve:

Manufacturer:	Velan Valve Manufacturing Co.
Size:	3"
Rating:	1500#
Type:	Manual Gate
Drawing:	88406
Bolting Material:	A193 GRB7
Bolting Size:	5/8-11 UNC x 4 5/8"

The degradation of the bolting was evaluated on April 28 and determined to be reporable in accordance with Technical Specification 15.6.9.2.A.3 as an abnormal degradation discovered in the reactor coolant pressure boundary.

The evaluation of the affected studs determined that the extent of the boric acid wastage was as follows:

Stud	Original Diameter	Depth of Wastage
1	5/8"	1/16"
2	5/8"	5/32"
3	5/8"	5/32"
4	5/8"	5/16"

One additional valve of similar design was examined with no indication of boric acid wastage of the bolting noted. The additional valve examined was RC-559B, the "B" loop RTD bypass line return isolation valve (3" Velan gate valve). Two other similar valves, RC-543, "A" loop decon connection isolation valve (4" Velan gate valve), and RC-544, "B" loop decon connection isolation valve (4" Velan gate valve), will be examined for evidence of boric acid wastage during this refueling outage. These examinations will be conducted with the bolting in place.

To allow easy inspection of the bolting on RC-559A and RC-559B and to aid in future leak detection removable insulation will be installed on these valves.

For the past six months, there has been a slight increase in containment airborne particulate activity. It is believed that the body-to-bonnet leak on RC-559A may have contributed to this.

The health and safety of the public was not endangered by this occurrence.

The NRC Resident Inspector has been notified of this event.

Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

May 11, 1983

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-301
LICENSEE EVENT REPORT NO. 83-005/01T-0
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report No. 83-005/01T-0 (a 14-day follow-up report) with an attachment which provides a description of an event reportable in accordance with Technical Specification 15.6.9.2.A.3, "Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment."

Very truly yours,

C W Fay

Vice President-Nuclear Power

C. W. Fay

Enclosure

Copy to NRC Resident Inspector

IE22

MAY 13 1983