

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | I | L | Q | A | D | I | 2 | 0 | 0 | 0 | - | 0 | 0 | 0 | - | 0 | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | _____ | 5
7 8 9 14 15 25 26 30 57 CAT 58

CON'T
01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 5 | 4 | 7 | 0 | 4 | 1 | 2 | 8 | 3 | 8 | 0 | 5 | 1 | 0 | 8 | 3 | 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
02 | At 1330 hours, on April 12, 1983, the 1-1001-65C Residual Heat Removal (RHR) Service
03 | Water pump was isolated and taken out of service to facilitate the repair of a
04 | broken coupling on the drain line of the 1-5745-C RHR Service Water Vault Room
05 | Cooler. All surveillances which are required by Technical Specification 4.5.B.2
06 | were successfully performed prior to taking the pump out of service. The 'A' RHR
07 | Containment Cooling Loop, all ECCS Systems, and the Diesel Generators were all
08 | operable at this time. Therefore, safe Reactor operation was not affected.
7 8 9 80

09 | SYSTEM CODE | S | B | 11 | CAUSE CODE | E | 12 | CAUSE SUBCODE | C | 13 | COMPONENT CODE | P | I | P | E | X | X | 14 | COMP SUBCODE | A | 15 | VALVE SUBCODE | Z | 16
7 8 9 10 11 12 13 18 19 20
17 | LER/RO REPORT NUMBER | 8 | 3 | 21 | 22 | SEQUENTIAL REPORT NO. | 0 | 1 | 7 | 24 | 26 | OCCURRENCE CODE | 0 | 3 | 28 | 29 | REPORT TYPE | L | 30 | REVISION NO. | 0 | 32
ACTION TAKEN | A | 18 | FUTURE ACTION | Z | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 0 | 37 | ATTACHMENT SUBMITTED | N | 23 | NPRD-4 FORM SUB. | H | 24 | PRIME COMP. SUPPLIER | N | 25 | COMPONENT MANUFACTURER | X | 9 | 9 | 9 | 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10 | The 3/8 inch hydraulic hose coupling broke due to an over-pressurization of the
11 | piping when the pump was started. The high pressure was caused by the failure of
12 | the RHR heat exchanger discharge valve to open. This event is being reported in
13 | LER/RO 83-18/03L. A socket-weld coupling and a section of carbon steel piping was
14 | installed. The pump was returned to service and satisfactorily tested
7 8 9 80

15 | FACILITY STATUS | E | 28 | % POWER | 1 | 0 | 0 | 0 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | A | 31 | DISCOVERY DESCRIPTION | Operator Observation | 32
7 8 9 10 12 13 44 45 46 80

16 | ACTIVITY CONTENT RELEASED | Z | 33 | Z | 34 | AMOUNT OF ACTIVITY | NA | 35 | LOCATION OF RELEASE | NA | 36
7 8 9 10 11 44 45 80

17 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | N | 38 | DESCRIPTION | NA | 39
7 8 9 10 11 12 13 80

18 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | NA | 41
7 8 9 10 11 12 80

19 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | NA | 43
7 8 9 10 80

20 | PUBLICITY ISSUED DESCRIPTION | N | 44 | 8305240317 830510 PDR ADOCK 05000254 S PDR
7 8 9 10 68 69 80 NRC USE ONLY

NAME OF PREPARER: W Leaverton PHONE: 309-654-2241, ext 198



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Quad Cities Nuclear Power Station
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Telephone 309/654-2241

NJK-83-168

May 10, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One
Appendix A, Section 3.5.B.2

Enclosed please find Reportable Occurrence Report Number RO 83-17/03L-0
for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of
Technical Specification 6.6.B.2.b; condition leading to operation in a
degraded mode permitted by a limiting condition for operation.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
N. Chrissotimos
INPO Records Center

MAY 13 1983

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