FORM 366 UPDATE REPORT - PREVIOUS REPORT DATE 12-22-78 **U. S. NUCLEAR REGULATORY COMMISSION**

## LICENSEE EVENT REPORT

CONTROL BLOCK: ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 LICENSEE CODE: N E F C S 1 ② LICENSE NUMBER: 0 0 0 0 0 0 0 0 0 0 0 ③ LICENSE TYPE: 4 1 1 1 1 ④ CAT ⑤

CON'T 0 1 REPORT SOURCE: L ⑥ DOCKET NUMBER: 0 5 0 0 0 2 8 5 ⑦ EVENT DATE: 1 2 0 5 7 8 ⑧ REPORT DATE: 0 1 0 9 7 9 ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 During the performance of the type B and C containment leak rate tests during the

0 3 1978 refueling outage, it was noted that the total leakage exceeded the 0.6 La limit

0 4 of 62,951 sccm.

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0 7

0 8

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|---|---|---|---|---|---|---|---|---|
| 0 9 | SYSTEM CODE | CAUSE CODE | CAUSE SUBCODE | COMPONENT CODE | COMP. SUBCODE | VALVE SUBCODE | | |
| | S D ⑪ | E ⑫ | B ⑬ | V A L V E X ⑭ | X ⑮ | D ⑯ | | |

| | | | | | | | |
|---|---|---|---|---|---|---|---|
| ⑰ LER/RO REPORT NUMBER | EVENT YEAR | | SEQUENTIAL REPORT NO. | | OCCURRENCE CODE | REPORT TYPE | | REVISION NO. |
| | 7 8 | — | 0 4 0 | / | 0 3 | L | — | 1 |

| | | | | | | | | |
|---|---|---|---|---|---|---|---|---|
| ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUTDOWN METHOD | HOURS ㉒ | ATTACHMENT SUBMITTED | NPRD-4 FORM SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER |
| D ⑱ | Z ⑲ | Z ⑳ | Z ㉑ | 0 0 0 0 | Y ㉓ | N ㉔ | A ㉕ | X 9 9 9 ㉖ |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 The penetrations that exhibited excess leakage were repaired and the total containment

1 1 leakage was reduced to below the 0.6 La (62,951 sccm) limit. A modification to the con-

1 2 tainment purge valves was completed this refueling outage which should reduce the leak-

1 3 age attributable to these valves, even if after repeated cycling. Quarterly leakage

1 4 tests will continue to be performed on the containment purge valve penetrations.

| | | | | | |
|---|---|---|---|---|---|
| 1 5 | FACILITY STATUS | % POWER | OTHER STATUS ㉚ | METHOD OF DISCOVERY | DISCOVERY DESCRIPTION ㉜ |
| | H ㉘ | 0 0 0 ㉙ | Refueling Shutdown | B ㉛ | Refueling leak rate test |

| | | | | |
|---|---|---|---|---|
| 1 6 | ACTIVITY RELEASED | CONTENT OF RELEASE | AMOUNT OF ACTIVITY ㉟ | LOCATION OF RELEASE ㊱ |
| | Z ㉝ | Z ㉞ | NA | NA |

PERSONNEL EXPOSURES

1 7 NUMBER: 0 0 0 ㊲ TYPE: Z ㊳ DESCRIPTION ㊴: NA

PERSONNEL INJURIES

1 8 NUMBER: 0 0 0 ㊵ DESCRIPTION ㊶: NA

LOSS OF OR DAMAGE TO FACILITY ㊸

1 9 TYPE: Z ㊷ DESCRIPTION: NA

PUBLICITY

2 0 ISSUED: N ㊹ DESCRIPTION ㊺: NA

NRC USE ONLY

7901170208

NAME OF PREPARER J. Connolley/G. Peterson PHONE: 402-426-4011

LER 78-040 Revision 1
Omaha Public Power District
Fort Calhoun Station Unit No. 1
Docket No. 05000285

Attachment No. 1

Safety Analysis

The air leakage tests performed during the 1978 refueling outage indicated a total leakage rate of 481,304.27 sccm. Of this leakage, however, only 183,288.07 sccm can be attributed to direct containment atmosphere leakage. Since the containment purge valves were the largest contributors to this leakage, 143,002.17 sccm, and the leakage of the purge valves was verified to be 8,410 sccm on September 14, 1978, prior to cycling to purge containment after the shutdown, it can, therefore, be concluded that containment integrity was not violated during plant operation since the total leak rate to atmosphere would have been 48,695.9 sccm. The purge valves were repaired during the 1978 refueling outage as were other valves and mechanical seals to bring the total leak rate within the 0.6 La limit. A special report (as required by Technical Specification 5.9.3.e) will be submitted within 90 days of the leak test completion date.

RCAndrews