. \* \* 1 (12-81) 10 CFR 50 LICENSEE EVENT REPORT 3150-0011 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: (1)A L B R F 3 0 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 0 5 4 LICENSE TYPE 30 5 0 1 CONT REPORT L 6 0 5 0 0 0 2 9 6 0 2 0 9 8 3 0 0 3 0 7 8 3 9 SOURCE 50 0 CKET NUMBER 50 50 EVENT DATE 74 75 REPORT DATE 50 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During normal operation while performing daily source and background check of 0 2 CAM 3-RM-90-250 (Reactor and Turbine Building Vent Monitor), the sample pump was 03 0 4 discovered not operating. TS 3.8.B.8 requirements were not met for a period of 0 5 up to 24 hours due to a faulty control room flow disturbance alarm. The sample 0 6 pump was verified operable during the last daily check. No alarms were received on local radiation monitors during this event. There are no redundant systems 0 7 0 8 There was no effect on the health or safety of the public. 80 SYSTEM CAUSE CAUSE COMP. VALVE COMPONENT CODE NS TRU(14) MICI X (12) Z | (13) X (15) II Z (16) 0 9 12 19 SEQUENTIAL REPORT NO. CODE REPOR REVISION NO. LER/RO REPORT NUMBER 1 813 0 9 0 3 L 0 28 24 30 32 31 SHUTDOWN HOURS 22 ATTACHMENT NPRD-4 SUBMITTED FORM SUB. NPRD-4 PRIME COMP. ACTION FUTURE EFFECT MANUFACTURER 26 AKEN ACTION ON PLANT METHOD SUPPLIER N 24 0000 V 20 | Z (20) A 18 Z 19 Z 21 L 25 9999 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The event was caused by a blown fuse. The bus ABC 15 fuse was replaced. 1 0 The cause of the blown fuse is not known, but is considered a random event. 1 The control room alarm malfunction was caused by a faulty Allied RS5D relay. 2 Functional SI 4.8.B.4.3 is being revised to ensure that this relay is operating 1 3 4 properly. PACILITY 80 METHOD OF (30) NA TUS DISCOVERY DISCOVERY DESCRIPTION (32) Routine Inspection (31) 5 10 80 CONTENT (35) RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE (36) Z 33 Z 34 NA NA 6 80 DI DI DI TI 80 UMBER DESCRIPTION 0 0 0 0 NA 8 11 12 80 LOSS OF OR DAMAGE TO PACILITY Z (42) NA 9 ..... ISSUED DESCRIPTION (45) NRC USE ONLY 20 N 44 NA 8303150309 830307 PDR ADOCK 05000296 PDR Donald W. Norwood (205) 729-0621 80 HONE ....

. Tennessee Valley Authority Browns Ferry Nuclear Plant

Form BF 17 BF 15.2 2/12/82

## LER SUPPLEMENTAL INFORMATION

BFRO-50-296 /83009 Technical Specification Involved 3.8.8.8 Reported Under Technical Specification 6.7.2.b.(2) \* Date Due NRC 3/9/83

## Event Narrative:

Unit 1. was in a maintenance outage; unit 2 was in a refueling outage; and unit 3 was operating at 100-percent power. Only unit 3 was affected by the event.

During performance of a routine source and background check on continuous air monitor (CAM) 3-RM-90-250, a chemical lab analyst discovered that the CAM sample pump was not operating. The pump had been verified operating on the previous day. Technical Specification 3.8.B.8 requires that the reactor and turbine building vents be continuously monitored. These requirements were not met for a period of up to 24 hours due to a malfunction in the control room flow disturbance alarm.

Upon finding the CAM inoperable, Surveillance Instruction (SI) 4.8.B.1.A.2 (Airborne Effluent Release Rate by Manual Sampling) was initiated to ensure no release limits were exceeded. During the time the CAM was inoperable, there were no airborne contamination zones in use and there were no alarms received on local radiation monitors on unit 3. There was no effect on the health or safety of the public.

The cause of this event was a blown fuse. The fuse was replaced and the CAM returned to service. The cause of the blown fuse cannot be determined, but is considered a random event. The control room alarm malfunction was caused by a faulty relay. Functional SI 4.8.B.4.3 is being revised to ensure that this relay is periodically checked to ensure it is operating properly. This revision should be complete by April 1, 1983.

\* Previous Similar Events:

BFR0-50-296/82063

\*Revision:

Retention: Period - Lifetime; Responsibility - Document Control Supervisor