



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-40
OMAHA PUBLIC POWER DISTRICT
FORT CALHOUN STATION, UNIT NO. 1
DOCKET NO. 50-285

Background

The licensee for the Fort Calhoun Station, Omaha Public Power District, requested changes to the plant's Technical Specification on Reactor and Steam Generator Coolant Radioactivity. The correspondence with the staff covered the period from November 1980, to February 3, 1983. The proposed specification submitted with the February 3, 1983 letter is evaluated below.

Evaluation

The proposed Technical Specification on Reactor Coolant Radioactivity conforms to the provisions of the Standard Technical Specifications (STS) for Combustion Engineering plants. The levels specified are identical to the STS, providing for an equilibrium value and iodine spiking. The cumulative allowable operating time above the equilibrium value is 100 hours, versus 48 in the STS, but the total time in one 12 month period is retained as 800 hours. This difference is justified, based on the capacity of the plant's cleanup system. Retention of the same total time in a 12 month period still provides that the maximum conditional probability of a steam generator tube rupture accident from an elevated iodine condition will be the same as the STS, or about 0.1.

The licensee has adopted surveillance requirements that are not inconsistent with those in the STS. For example, the change from STS requirements for obtaining a coolant sample every four hours while above the equilibrium iodine value to one sample every eight hours is acceptable because, with the low capacity cleanup system, the time behavior of any iodine spike will be extended relative to what was assumed in the STS definition.

The licensee has adopted limits and surveillance requirements on the Steam Generator Coolant Radioactivity that are in conformance with the STS.

The staff concludes that adoption of the Technical Specification on limits and surveillance requirements on Reactor and Steam Generator Coolant Radioactivity will provide adequate mitigating features for Steam Generator Tube rupture and secondary side rupture accidents.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: February 23, 1983

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