

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

C
A
S
0
S
9
2
0
0
-
0
0
0
0
0
0
-
0
0
3
4
1
1
1
1
1
4
5

1
L
6
0
5
0
0
0
3
6
2
7
0
1
2
3
8
3
8
0
2
2
2
8
3
9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

2 On January 23, 1983 at 0330 while in Mode 3, pressure indicator 3PI-8300 for Steam
3 Generator E-088 was declared inoperable when it failed its Technical Specification
4 Surveillance per Section 4.3.3.5. LCO 3.3.3.5 was invoked. The Action Statement
5 associated with this LCO allows operations to continue up to 7 days before-requiring the
6 indicator to be returned to service or initiation of cooldown. Since the indicator was
7 returned to service within the allowable time constraints, there was no impact
8 on plant operation or the health and safety of plant personnel or the public.

3
1
F
11
A
12
C
13
V
A
I
V
E
X
14
X
15
D
16

8
3
0
1
1
0
3
L
0

X
19
X
9
4
Z
21
0
0
0
0
Y
23
N
24
N
25
X
9
9
9
9
25

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

5 The cause of this occurrence was inadvertent isolation of 3PI-8300 from its
6 associated detector, 3PT-8300 (see attachment). The valve lineup was corrected
7 and the indicator was declared operable in accordance with Procedure S023-3-3,28
8 on 1/28/83. This was an isolated occurrence. However, an evaluation is being performed
9 by Station Tech. Dept. to determine more appropriate Surveillance Test acceptance criteria
0 and valve lineup procedural requirements.

B
0
0
0
NA
B
11
Surveillance
Testing

Z
33
Z
34
NA
NA

0
0
0
Z
37
NA

0
0
0
NA

Z
42
NA

NA

8303110342 830222
 PDR ADOCK 05000362
 S PDR

NRC USE ONLY

H. B. RAY
 714/492-7708

ATTACHMENT TO LER 83-011
SOUTHERN CALIFORNIA EDISON COMPANY
SAN ONOFRE NUCLEAR GENERATING STATION
UNIT NO. 3, DOCKET NO. 50-362

SUPPLEMENTAL INFORMATION FOR CAUSE DESCRIPTION

It is believed that isolation of 3PI-8300 could have occurred during calibration of the detector and indicator performed on December 10, 1982, even though it successfully passed surveillance testing performed on January 3, 1983, prior to Mode 3 entry. During the January 3, 1983 surveillance, steam generator pressure was 90 psia and 3PI-8300 read 0, which is within the acceptance criterion of ± 120 psia.