(7-77)	RM 366 U. S. NUCLEAR REGULATORY COMMISSION
	LICENSEE EVENT REPORT
	CONTROL BLOCK:
0 1	N J S G S 1 2 0 <th0< th=""> <th0< th=""> <th0< th=""></th0<></th0<></th0<>
	REPORT L 6 0 5 0 0 2 7 2 0 0 2 1 5 8 3 8 0 3 0 1 8 3 9 SOURCE 60 61 DOCKET NUMBER 08 69 EVENT DATE 74 75 REPORT DATE 80
02	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On February 14, 1983, during startup of Salem Unit 1, results of zero power physics
03	testing revealed that the calculated Moderator Temperature Coefficient (MTC) value for
04	all control rods withdrawn was more positive than allowed by the Technical Specifications
05	With the completion of testing on February 15, 1983, Action Statement 3.1.1.4a applied.
06	Control rod withdrawal limits were calculated and implemented that day. The event is
07	reportable in accordance with Technical Specification 6.9.2.
08	9
09 78	SYSTEM CAUSE CODE CODE SUBCODE COMPONENT CODE SUBCODE
[T]0]	Image: Construct and the point of the p
111	Technical Specification limits; the calculated value is within expected deviation
12	of the design value. Limitation on control rod withdrawal will be maintained up
13	to a burnup of 500 MWD/MTU when the MTC value is expected to return to within
14	specification.
	9 30 METHOD OF DISCOVERY DESCRIPTION (32) 0 0 0 0 0 0 10 12 13 44
	CTIVITY CONTENT ELEASED OF RELEASE AMOUNT OF ACTIVITY 35 2 33 2 34 N/A LOCATION OF RELEASE 36 N/A N/A 80 PERSONNEL EXPOSURES 64
1 7 7 8	NUMBER 37 Z 38 N/A 9 PERSONNEL INJURIES 13 80
18	
7 8 1 9 7 8	9 11 12 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION 9 10 9 10 PUBLICITY (45) N/ B303140583 B30302 PDR ADOCK 05000272 NRC USE ONLY
20	SSUED DESCRIPTION 45 N/A PDR NRC USE ONLY 9 10 68 69 80 5
0	R. Frahm PHONE (609) 935-6000 Ext. 3078 0