2 271 LICENSEE EVENT REPORT . PLEASE PRINT OR TYPE ALL REQUIRED INFORMATIONS IIII0 CONTROL BLOCK: 0 0 0 0 0 0 0 0 5 3 0 2 0011238300222283 16 10 10 10 15 10 3 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On January 23, 1983 at 0330 while in Mode 3, pressure indicator 3PI-8301 for Steam 2 2 Generator 6-089 was declared inoperable when it failed its Technical Specification 111 surveillance per section 4.3.3.5. LCO 3.3.3.5 was invoked. The Action Statement 34 associated with this LCO allows operations to continue up to 7 days before-requiring : 5 the indicator to be returned to service or initiation of cooldown. Since the indicator 16 was returned to service within the allowable time constraints, there was no impact on 3 7 plant operation or the health and safety of plant personnel or the public. 2 8 1 1 con COOR CAUSE COMPONENT CODE ALL IV IE X D (16 00 IIF A (12) 3 9 REVISION SEQUENTAL ALPOAT NO. OCCURRENCE 0 11 10 0131 8 COMPONENT TTED PLIER 91 91 (24 (27) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS occurrence was inadvertent isolation of 3PI-8301 from its associated 1 The cause this detector, 3PT-8301 (see attachment). The valve lineup was corrected and the indicator j was declared operable in accordance with Procedure S023-3-3.28 on January 28., 1983. This was an isolated occurrence. However, an evaluation is being performed by Station Tech 3 Dept. to determine more appropriate Surveillance Test acceptance criteria and valve lineup 4 procedural requirements. HETHOD OF OTHER STATUS (30) DESCRIPTION (32) B (3) Surveillance 010 5 10 CON TENT OF ACTIVITY 35 LOCATION OF PELEASE (36) NA 6 19 I PORL IN FR DESCRIPTION 39 (3) 231.8 DESCRIPTION 0 0 4 (40) HACE TO FACILITY (43 8303110337 830222 NA Z (42 PDR ADOCK 05000362 9 PDR PULL OTY NAC USE ONLY DESCRIPTION (45 NA 6.8 83 714/492-7700 2

## ATTACHMENT TO LER 83-010 SOUTHERN CALIFORNIA EDISON COMPANY SAN ONOFRE NUCLEAR GENERATING STATION UNIT NO. 3, DOCKET NO. 50-362

## SUPPLEMENTAL INFORMATION FOR CAUSE DESCRIPTION

It is believed that isolation of  $3PI-83\cup1$  could have occurred during calibration of the detector and indicator performed on December 10, 1982, even though it successfully passed surveillance testing performed on January 3, 1983, prior to Mode 3 entry. During the January 3, 1983 surveillance, steam generator pressure was 90 psia and 3PI-8301 read 0, which is within the acceptance criterion of  $\pm$  120 psia.