



Portland General Electric Company

Bart D. Withers Vice President

March 4, 1983

Trojan Nuclear Plant
Docket 50-344
License NPF-1

Mr. Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington DC 20555

Dear Mr. Eisenhut:

TROJAN NUCLEAR PLANT
NUREG-0737, TMI-2 Action Item II.B.2 - Plant Shielding

PGE has provided the results of the radiation shielding review conducted in accordance with NUREG-0737, Item II.B.2, in letters to the NRC dated January 2, 1980, April 15, 1980, January 2, 1981, and April 28, 1982. This analysis was based on emergency instructions in place at Trojan as of January 1, 1980. One of the basic premises contained in the analysis was to prevent the spread of highly radioactive liquids or gases outside of the Containment in the event of severe core damage. That philosophy was implemented by the addition of a procedural precaution in Emergency Instruction EI-1 Rev. 11 (Loss of Reactor Coolant) to not let down reactor coolant from Containment in the event of severe core damage.

Subsequent to our original shielding review, PGE has implemented revised emergency instructions at Trojan based on the Westinghouse Owners Group Emergency Response Guidelines. As referenced in PGE letter to the NRC dated October 27, 1982, PGE has reperformed the radiation shielding review based on emergency instructions implemented to date. This review has confirmed that the assumptions made during the original shielding analysis are still valid.

The philosophy of preventing the spread of highly radioactive liquid or gases outside the Containment has been extended to the new emergency instructions. To accommodate the new procedure format, Functional Restoration Instruction C-1 (Response to Inadequate Core Cooling) will be revised to include general instructions preventing letdown of reactor coolant in the event of severe core damage. This procedure revision will be implemented prior to the end of the current Plant outage.

8303090218 830304
PDR ADOCK 05000344
P PDR

121 S.W. Salmon Street Portland, Oregon 97204

ADK

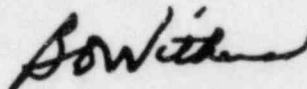
Portland General Electric Company

Mr. Darrell G. Eisenhut, Director
March 4, 1983
Page two

One clarification to the previous assumptions is discussed below. The original evaluation assumed that the Reactor Coolant System remained on natural circulation for seven days prior to initiating use of the Residual Heat Removal (RHR) System to allow decay of short-lived isotopes. This assumption allowed access to the hallways on Elevations 5 and 25 of the Trojan Auxiliary Building, where the RHR System is located. It has been subsequently determined that there are no reasons to require access to these hallways and thus this restriction is no longer necessary.

Finally, there was a typographical error in PGE letter to the NRC dated April 28, 1982. On Attachment 1 of Page 1, Item 2A, the text should read "Installation of valve reach rods for draining the waste gas surge tank . . ." versus the waste gas decay tank.

Sincerely,



Bart D. Withers
Vice President
Nuclear

c: Mr. Lynn Frank, Director
State of Oregon
Department of Energy

Mr. R. H. Engelken
Regional Administrator
U. S. Nuclear Regulatory Commission
Region V