LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.7.A.5 Oxygen Concentration	4.7.A.5 Oxygen Concentration
a. After completion of the startup test program and demonstration of plant electrical output, the primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as stated in Specification 3.7.A.5.b.	The primary containment oxygen concentration shall be measured and recorded daily in the main control room.
b. Within the 24-hour* period subsequent to placing the reactor in the Run Mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. Deinerting may commence 24 hours prior to a shutdown.	*A 72-hour period is allowed for the startup in progress on February 7, 1982. *A 72-hour period is allowed for the startup in progress on February 22, 1983.
6. Containment Atmosphere Dilution (CAD)	6. Containment Atmosphere Dilution (CAD)
a. Operability Requirements	a. Functional Test
After completion of the startup test program and demonstration of plant electrical output and thereafter whenever the reactor is in power operation, the post-LOCA containment Atmosphere Dilution (CAD) System must be operable and capable of supplying nitrogen to the primary containment for dilution if required by post-LOCA conditions. If this specification cannot be met, the system must be restored to an operable condition within seven days or the reactor must be taken out of power operation.	The post-LOCA Containment Atmosphere Dilution (CAD) System shall be functionally tested once per operating cycle.
b. Seven-Day Nitrogen Supply	Seven-Day Nitrogen Supply
After completion of the startup test program and demonstration of plant electric output and thereafter whenever the reactor is in power operation, the CAD System shall contain a minimum of 2000 gallons of liquid nitrogen. If this specification cannot be met, the minimum volume will be restored within seven days or the reactor must be taken out of power operation.	The level in the liquid nitrogen storage tanks shall be recorded twice weekly.

3.7.A.6.c. H_2 and O_2 Analyzer

Whenever the reactor is in power operation, there shall be at least one CAD System H_2 and O_2 analyzer serving the primary containment. If one H_2 and O_2 analyzer is inoperable, the reactor may remain in operation for a period not to exceed seven days.

4.7.A.6.c. H_2 and O_2 Analyzer

Instrumentation surveillance is listed in Table 4.2-11.

d. Post-LOCA Repressurization Limit

The maximum post-LOCA primary containment repressurization limit allowable using the CAD System shall be 30 psig. Venting via the SGTS to the main stack must be initiated at 30 psig following the initial post-LOCA pressure peak.

7. Drywell-Suppression Chamber Differential Pressure

Differential pressure between the drywell and suppression chamber shall be maintained equal to or greater than 1.5 psid except as specified in (1) and (2) below: If this specification cannot be met, and the differential pressure cannot be restored within the subsequent six (6) hour period, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.

1) This differential pressure shall be established within 24 hours* after having placed the Mode Switch in the RUN mode. The differential pressure may be removed within 24 hours prior to achieving a shutdown.

2) This differential pressure may be decreased to less than 1.5 psid for a maximum of four hours during required operability testing of the HPCI system pump, the RCIC system pump, and the drywell-pressure suppression chamber vacuum breakers.

7. Drywell-Suppression Chamber Differential Pressure

The pressure differential between the drywell and suppression chamber shall be recorded once each shift.

*A 72-hour period is allowed for the start-up in progress on February 7, 1982.

*A 72-hour period is allowed for the start-up in progress on February 22, 1983.

ATTACHMENT 1

NRC DOCKET 50-321
OPERATING LICENSE DPR-57
EDWIN I. HATCH NUCLEAR PLANT UNIT 1
PROPOSED CHANGES TO TECHNICAL SPECIFICATIONS

The proposed change to Technical Specifications (Appendix A to Operating License DPR-57) would be incorporated as follows:

Remove Page	Insert Page
3.7-9	3.7-9
3.7-10	3.7-10