

# LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

V A S P S 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

REPORT SOURCE L 6 0 5 0 0 0 2 8 0 7 0 1 1 4 8 3 8 0 2 0 8 8 3 9

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

With the unit at full power, the "C" Steam Generator Level Indication LT-1-495 drifted high. This event is contrary to T.S. Table 3.7-1 and Table 3.7-2, and is reportable in accordance with T.S.6.6.2.b.(2). The redundant channels remained operable, therefore, the health and safety of the public were not affected.

SYSTEM CODE C H 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE I N S T R U 14 COMP. SUBCODE T 15 VALVE SUBCODE Z 16

LER/RO REPORT NUMBER 8 3 17 EVENT YEAR 8 3 18 SEQUENTIAL REPORT NO. 0 0 2 19 OCCURRENCE CODE 0 3 20 REPORT TYPE L 21 REVISION NO. 0 22

ACTION TAKEN X 18 FUTURE ACTION X 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 23 ATTACHMENT SUBMITTED Y 24 NRC FORM SUB. N 25 PRIME COMP. SUPPLIER L 26 COMPONENT MANUFACTURER R 3 6 9 27

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

The cause of the failure is believed to be a leak on the transmitter reference leg condensate pot Root Valve. Repairs will be performed during the next outage.

FACILITY STATUS E 28 % POWER 1 0 0 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator Observation 32

ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36

PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39

PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41

LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43

PUBLICITY ISSUED DESCRIPTION N/A 44

8302170371 830208  
PDR ADOCK 05000280  
S PDR

NRC USE ONLY

NAME OF PREPARED J. L. Wilson

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ATTACHMENT 1  
SURRY POWER STATION, UNIT NO. 1  
DOCKET NO: 50-280  
REPORT NO: 83-002/03L-0  
EVENT DATE: 01-14-83

TITLE OF THE EVENT: HIGH INDICATION ON STEAM GENERATOR LEVEL INSTRUMENTATION

1. Description of the Event

On January 14, 1983, with the unit at full power, channel 2 of "C" steam generator level indication (LI-1-495) drifted  $\approx$  5% higher than channels 1 or 3. The channel was declared inoperable and the bistables were placed in the trip mode. Since the channel failure was in the non conservative direction, the minimum degree of redundancy as required by Technical Specification Table 3.7-1 and 3.7-2 was not available. This event is contrary to Tech. Spec. 3.7.B and is reportable in accordance with T.S.6.6.2.b(2).

2. Probable Consequences and Status of Redundant Equipment

Steam Generator Level Channel 2, LI-1-495, provides protection functions. Automatic reactor trip initiation will occur when 1 of 2 channels on any steam generator indicates Low Water Level coincident with a steam flow/feed flow mismatch, or 2 of 3 Low Low Steam Generator Level. Since the redundant channels remained operable, the health and safety of the public were not affected.

3. Cause

The cause of the failure is believed to be a leak on the transmitter reference leg condensate pot Root Valve.

4. Immediate Corrective Action

The redundant channels of level instrumentation were verified operable and the bistables for LI-1-495 were placed in trip.

5. Subsequent Corrective Action

Repairs will be performed during the next cold shutdown of the unit.

6. Action Taken to Prevent Recurrence

None at this time.

7. Generic Implications

None.