



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 81 TO FACILITY LICENSE NO. DPR-34

PUBLIC SERVICE COMPANY OF COLORADO

FORT ST. VRAIN NUCLEAR GENERATING STATION

DOCKET NO. 50-267

1.0 INTRODUCTION

By letter dated January 25, 1990, Public Service Company of Colorado, (PSC), requested changes to the Technical Specifications (TS), to reflect administrative changes to the management organization at the Fort St. Vrain Nuclear Generating Station (FSV).

PSC is proposing TS changes in four areas:

- (a) The title Senior Vice President, Nuclear Operations is changed to Vice President, Nuclear Operations.
- (b) The duties of the Nuclear Support Manager and the Nuclear Training Manager have been combined. The new title for the person responsible for these duties is Nuclear Training and Support Manager.
- (c) The Nuclear Training and Support Manager is being added to the Nuclear Facility Safety Committee (NFSC).
- (d) The address for the Annual Radiological Environmental Report is being revised to be consistent with address requirements of 10 CFR 50.4.

2.0 EVALUATION

FSV was shut down on August 18, 1989 because of control rod failures. The shutdown was made permanent because of subsequent discovery of degradation of steam generator ring headers. PSC began defueling on November 27, 1989 and completed the removal of one-third of the core (the maximum capacity of its on site fuel storage wells) on February 7, 1990. On February 2, 1991, PSC began construction of an independent spent fuel storage installation for storage of all of the FSV spent fuel.

All of the proposed changes in the TS are administrative and none of the changes reduce safety requirements for FSV.

The change in the title from Senior Vice President Nuclear Operations to Vice President, Nuclear Operations does not change the authority, the duties, or the responsibilities of the individual involved. Therefore, there is no decrease in safety requirements.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction in chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility authorized herein.

D. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter 1: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and the appropriate sections of Parts 70 and 73; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 842 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 81, are hereby incorporated in the license. The licensee shall maintain the facility in accordance with the Technical Specifications.

- (3) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: Fort St. Vrain Security Plan, Fort St. Vrain Guard Training and Qualification Plan, and Fort St. Vrain Safeguards Contingency Plan. All plans are with revisions submitted through June 6, 1990 as supplemented by letter dated September 14, 1990.

Combining the Nuclear Support Management duties with duties of the Nuclear Training Manager is consistent with the reduced work load at the permanently shutdown FSV. This change does not reduce the TS job duties requirements and is therefore acceptable. Also, this individual has been added to the NFSC which provides another input into this safety committee that increases and improves its input on important safety issues. We therefore, find this change acceptable. Changing the TS address for submittal of the Annual Radiological Environmental Monitoring Report does not impact on safety issues for FSV.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Colorado State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping or administrative procedures or requirements. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (55 FR 12599). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSIONS

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: April 5, 1991

7.1 ORGANIZATION, REVIEW, AND AUDIT-ADMINISTRATIVE CONTROLS

Applicability

Applies to the lines of authority and responsibility for the operational safety of the facility, and the organization for periodic review and audit of facility operation.

Objectives

To define the principal lines of authority and responsibility for providing continuing review, evaluation, and improvement of the plant operational safety.

SPECIFICATION AC 7.1.1 - ORGANIZATION, ADMINISTRATIVE CONTROLS

1. RESPONSIBILITY

- a. The Manager, Nuclear Production and Station Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.
- b. The Shift Supervisor (or during his absence from the control room and Shift Supervisor's office, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Vice President, Nuclear Operations shall be reissued to all station personnel on an annual basis.

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(3) The Vice President, Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

(4) The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

b. Unit Staff

(1) Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 7.1-1.

(2) A licensed operator must be in the control room at all times when fuel is in the reactor.

(3) During shutdown and refueling conditions, an individual (other than the Technical Advisor) with a valid RO (or SRO) license shall be present in the control room. During all other conditions, an individual (other than the Technical Advisor) with a valid SRO license shall be present in the control room.

5. TRAINING

- | a. A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training and Support Manager, and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR 55.
- | b. A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training and Support Manager, and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training/drill sessions which shall be held at least once per calendar quarter.
- | c. The Technical Advisors shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents. An initial training and retraining program for the Technical Advisors shall be maintained under the direction of the Nuclear Training and Support Manager. The Technical Advisors shall also receive training in plant design and layout, including the capabilities of instrumentation and controls in the control room.

Specification AC 7.1.2 - Plant Operations Review Committee
(PORC), Administrative Controls

The organization, responsibilities, and authority of the PORC shall be as follows:

1. Membership

The Plant Operations Review Committee shall be composed of the following:

Chairman: Manager, Nuclear Production and Station
Manager
Nuclear Training and Support Manager
Superintendent of Chemistry and Radiation Protection
(Radiation Protection Manager)
Superintendent of Operations
Maintenance Department Manager
Systems Engineering Manager
Superintendent of I&C Maintenance
Operations Manager

2. Alternates

An alternate chairman and alternate members, if required, shall be appointed in writing by the PORC Chairman to serve in the absence of a chairman or a member; however, no more than two alternate members shall participate as voting members in PORC activities at any one time.

- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical and Electrical Engineering
- h. Quality Assurance Practices
- i. Other appropriate fields associated with the unique characteristics of the nuclear power plant.

2. Membership

The NFSC shall be composed of the following:

Chairman (As appointed - See Step 4)

Manager, Nuclear Licensing and Resource Management Division

Manager, Nuclear Production and Station Manager

Manager, Nuclear Engineering Division

Manager, Quality Assurance Division

Nuclear Training and Support Manager

Safety and Security Director

Consultants, as required, shall be appointed in writing by the Vice President, Nuclear Operations.

3. Alternates

Alternate members, if required, shall be appointed in writing by the Vice President, Nuclear Operations; however, no more than two alternate members shall participate as voting members in NFSC activities at any one time.

4. Chairman

The Chairman and Alternate Chairman of the NFSC shall be appointed in writing by the Vice President, Nuclear Operations and shall serve as members of the NFSC.

5. Consultants

Consultants shall be utilized as determined by the Chairman, NFSC, to provide expert advice to the NFSC.

6. Meeting Frequency

The NFSC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

7. Quorum

A quorum of the NFSC shall consist of the Chairman or his designated alternate and a majority of the NFSC members including alternates. No more than a minority of the quorum shall have line responsibilities for operation of the facility.

8. Responsibilities

a. The Nuclear Facility Safety Committee shall review:

- (1) The safety evaluations for safety significant changes to procedures, equipment, or systems

personnel or an outside fire protection firm;

- (b) a biennial audit of the fire protection program and implementing procedures;
 - (c) a triennial fire protection and loss prevention inspection and audit utilizing an outside qualified fire consultant.
- (9) The Offsite Dose Calculation Manual and Process Control Program and implementing procedures at least once per 24 months.
 - (10) The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months.
 - (11) The performance of activities required by the Quality Assurance Program to meet the provisions of Regulatory Guide 1.21 Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975, at least once per 12 months.

9. Authority

The NFSC shall report to and advise the Vice President, Nuclear Operations on those areas of responsibility specified in 8.a, 8.b and 8.c above.

10. Records

Records of NFSC activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each NFSC meeting shall be prepared and forwarded to the Vice President, Nuclear Operations within 30 days following each meeting.

- | b. After preliminary approval by the Vice President,
Nuclear Operations, the minutes shall be distributed
to all NFSC members, and approved at the next NFSC
meeting.

- | c. Reports of reviews encompassed by Section 8.a,
above shall be forwarded to the Vice President,
Nuclear Operations, within 30 days following
completion of the review.

- | d. Audit reports encompassed by Section 8.c, above
shall be forwarded to the Vice President, Nuclear
Operations, and to the management positions
responsible for the areas audited within 30 days
after completion of the audit.

Each monthly operating report shall include:

1. A narrative summary of operating experience during the report period relating to safe operation of the facility, including major safety-related maintenance.
2. Report of any single release of radioactivity or radiation exposure which accounts for more than 10% of the allowable annual values.
3. Indications of failed fuel resulting from irradiated fuel examinations, completed during the report period.
4. The monthly statistical information contained in Regulatory Guide 1.16.

d. Annual Radiological Environmental Monitoring Report

A report on the Radiological Environmental Monitoring Program for the previous calendar year shall be submitted to the Nuclear Regulatory Commission in accordance with 10 CFR 50.4 as a separate document by May 1 of each year.

The Annual Radiological Environmental Monitoring Reports shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental monitoring activities for the report period, including a comparison with pre-operational studies, operational controls (as appropriate), and previous environmental monitoring reports, and an assessment of the observed impacts of the plant operation on the environment. The reports shall also include the results of land-use censuses required by Specification ELCD 8.2.1.

The Annual Radiological Environmental Monitoring Reports shall include the results of analysis of all radiological environmental samples and of all measurements taken during the period pursuant to the Table and Figures in the ODCM, as well as summarized and tabulated results of these analyses and measurements in the format of the table in the Radiological Assessment Branch Technical Position, Revision 1, November, 1979. In the event that some individual results are not